

Neutron total Cross-sections and Resonance parameters of Dy at Pohang Neutron Facility

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2010 AASPP Workshop

**The 1st Asian Nuclear Reaction Database Development
Workshop**

Total Cross Section Measurement of Pohang Neutron Facility

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5. Signal and Noise separation
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7. Flight Length and t_0 fitting
8. Dysprosium FADC TOF Spectra

1. Total Cross Section Measurement

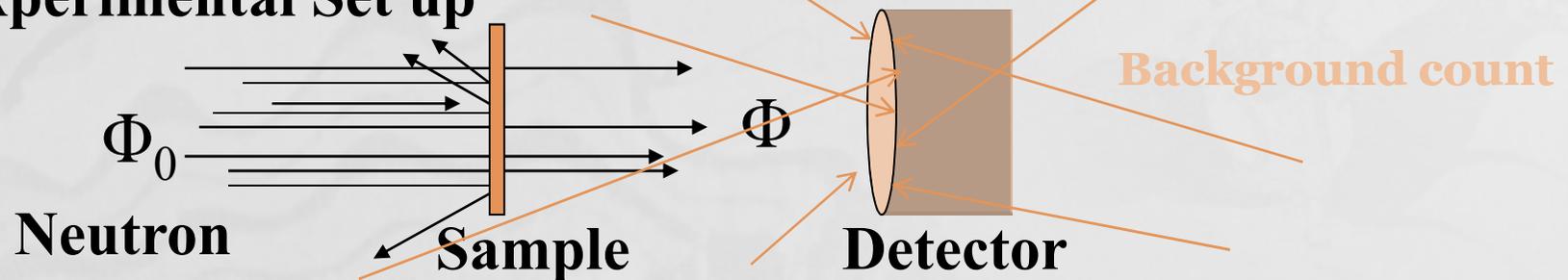
1. Neutron Energy E in eV corresponding to channel I in TOF

$$E[eV] = \left(\frac{72.3 \times L[m]}{T[\mu s]} \right)^2$$

L : flight path length

T : time of flight

2. Experimental Set up



3. Neutron Transmission rate

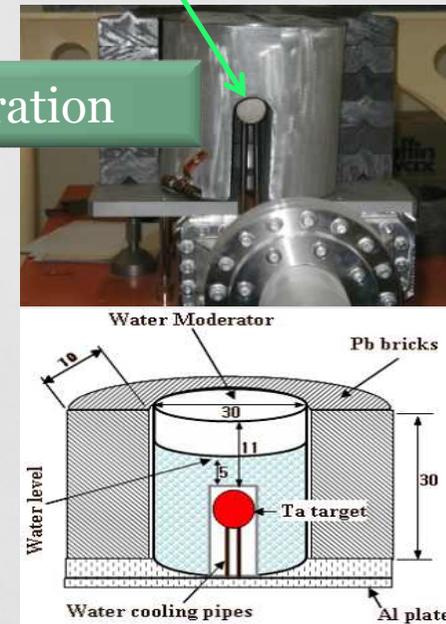
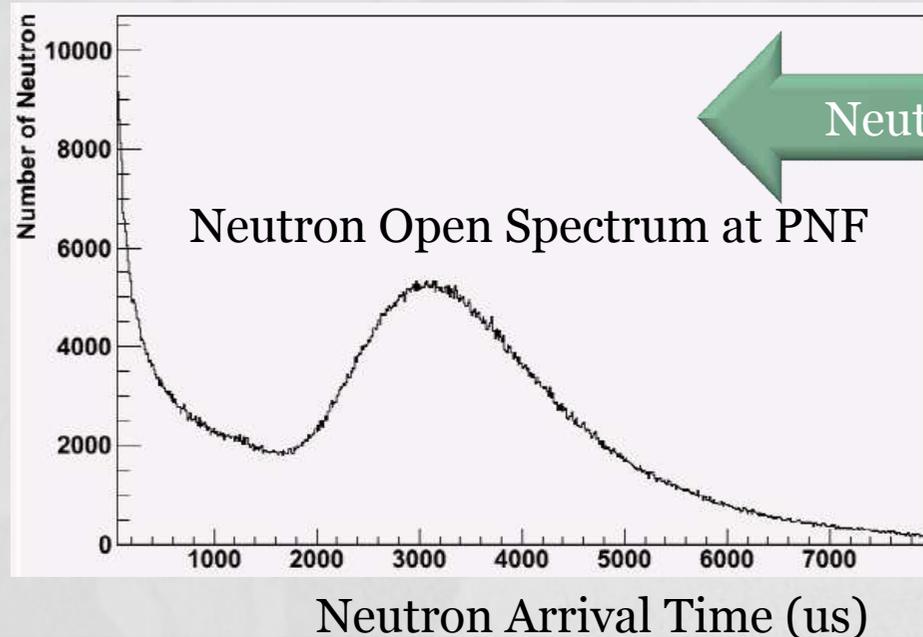
$$T(E_i) = \frac{\text{Count}_{\text{with Sample}}}{\text{Count}_{\text{without Sample}}} = \frac{\{In(E_i) - In^B(E_i)\} / M}{\{Out(E_i) - Out^B(E_i)\} / M^B} = \frac{C_{In}(E_i) / M}{C_{Out}(E_i) / M^B}$$

4. Total Cross Section

$$\sigma(E) = \frac{1}{N} \ln T(E)$$

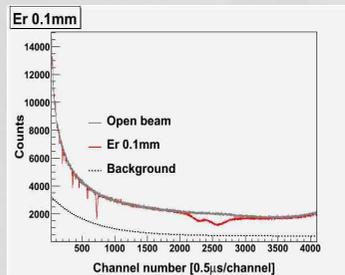
2. Pohang Neutron Facility

- ◆ Electron accelerator based Time of Flight system
 - ✓ electron energy = 50 ~ 70 MeV
 - ✓ repetition rate = Below 30Hz
 - ✓ pulse width = 1 ~ 2 μ s
 - ✓ peak beam current = 30 ~ 60 mA
 - ✓ TOF flight length = 11.5~12m
 - ✓
- ◆ Target + water moderator : to produce neutron pulse
 - ✓ Ta plates + cooling system



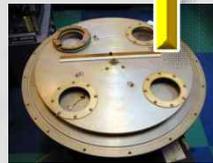
3. Current TOF DAQ

Total Cross-section



BC702
Neutron
Detector

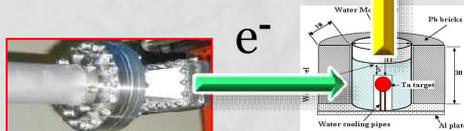
Neutron



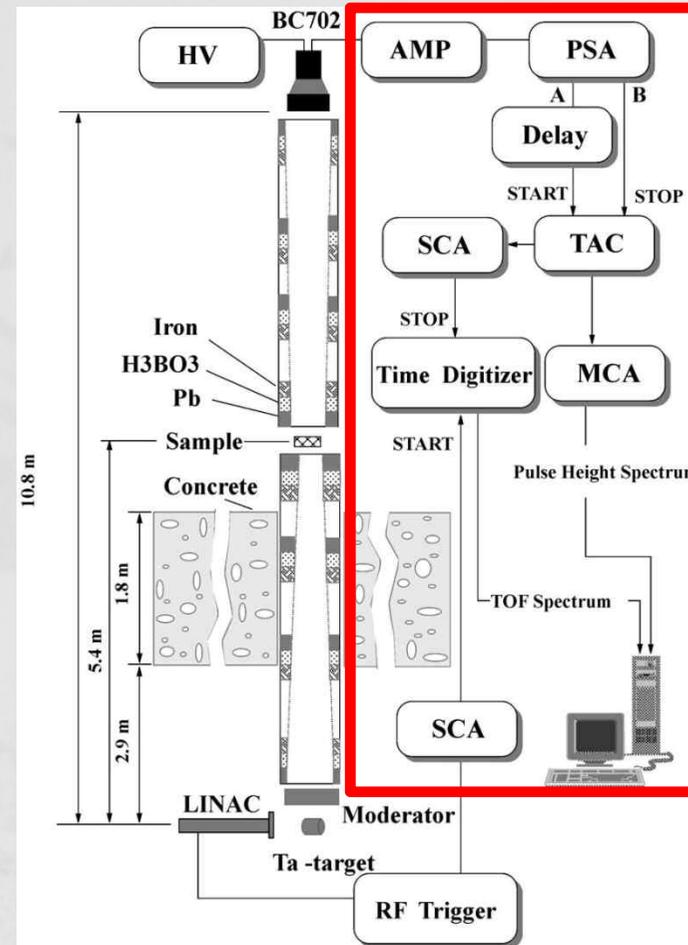
Sample
Changer

Neutron

Scattering,
Capture, etc

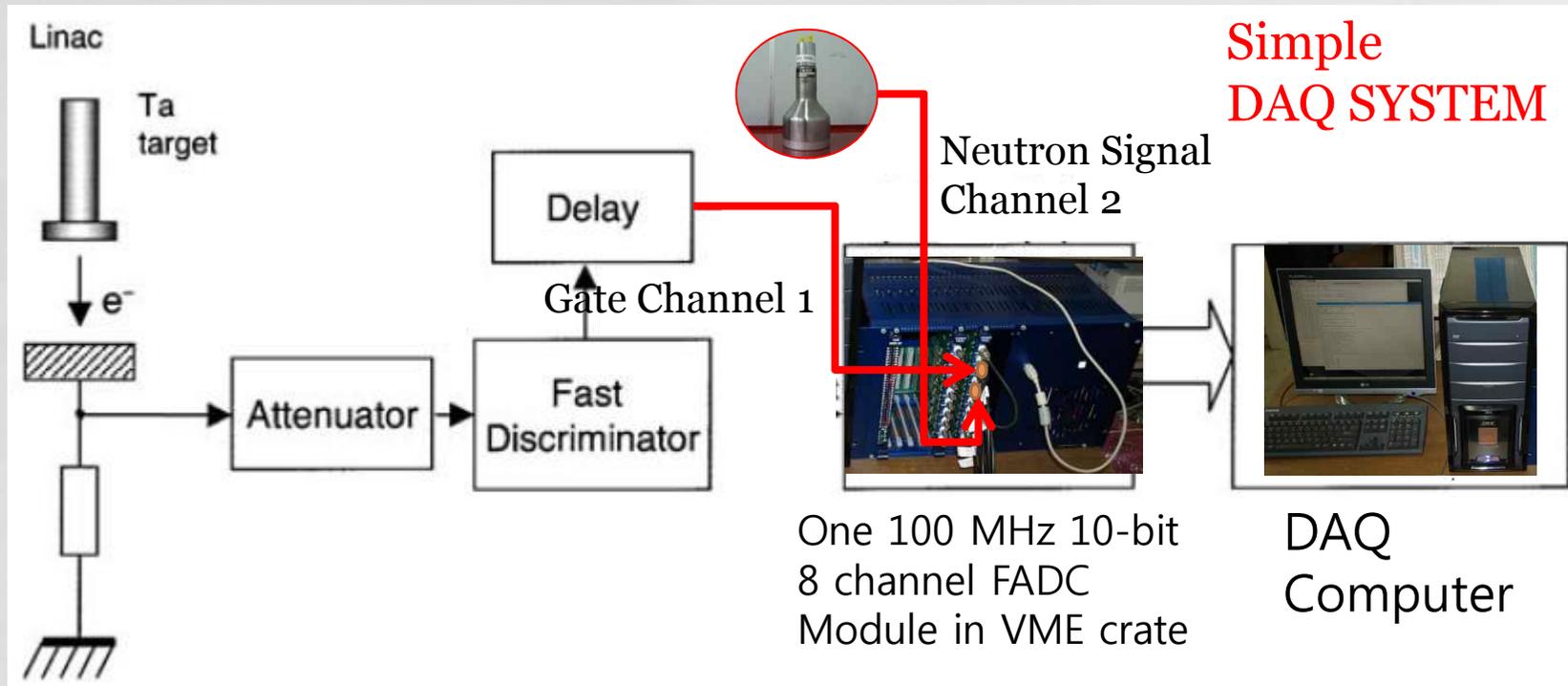


Current DAQ



FADC
DAQ

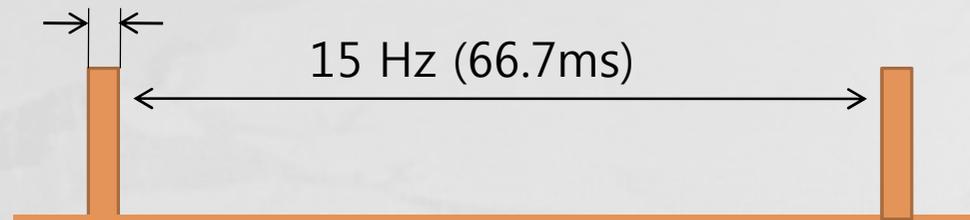
4. Flash ADC TOF DAQ (1)



- TOF Signal Data without Noise is possible due to the pulse shape analysis

4. Flash ADC TOF DAQ (2)

PAL 1~2 us



Beam Pulse

First Signal ~~~~

Last Signal

Noise



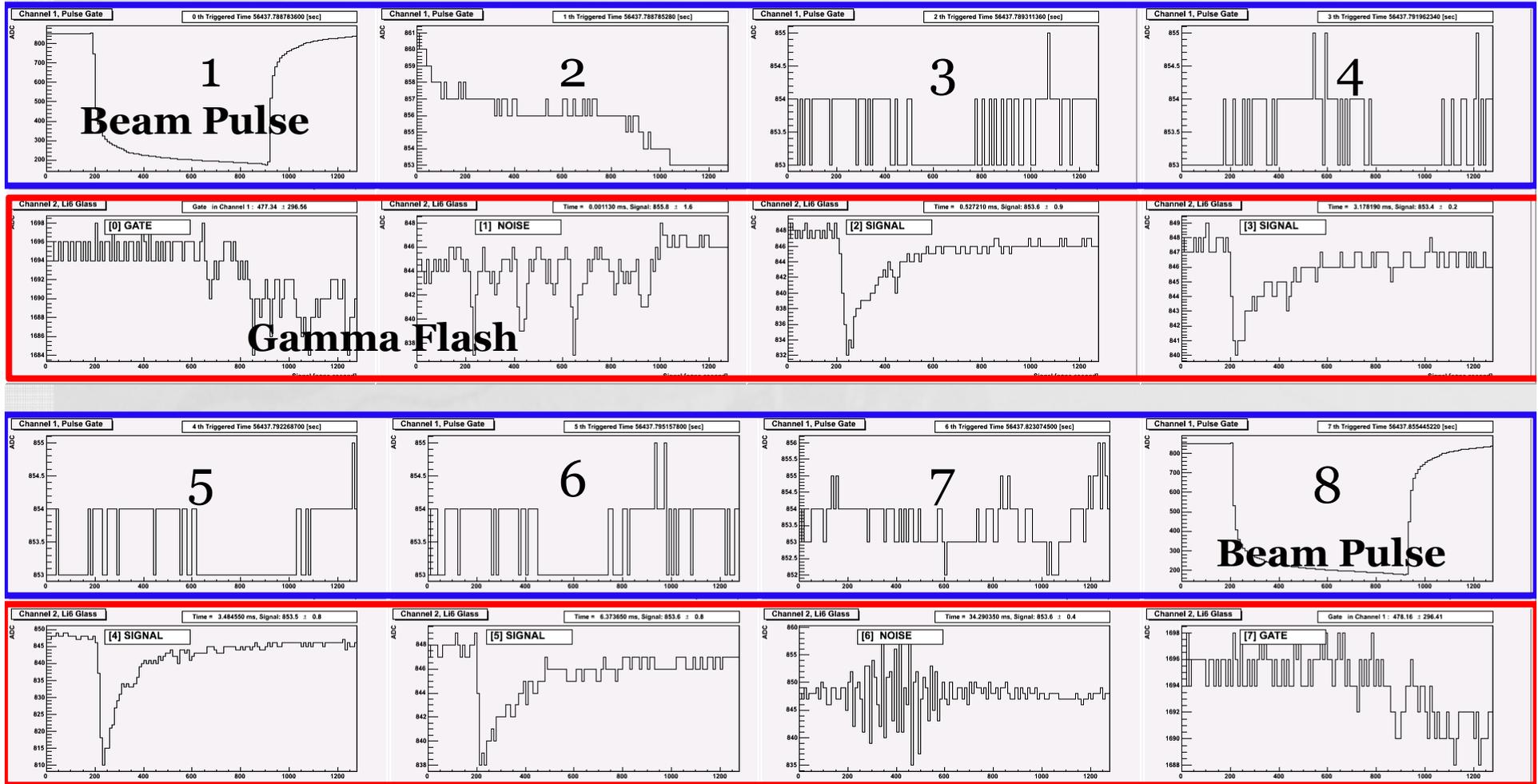
Gamma Flash due to Bremsstrahlung

Noise Rejection using a Software is possible and easy using FADC

4. Flash ADC TOF DAQ (3)

Recorded GATE channel

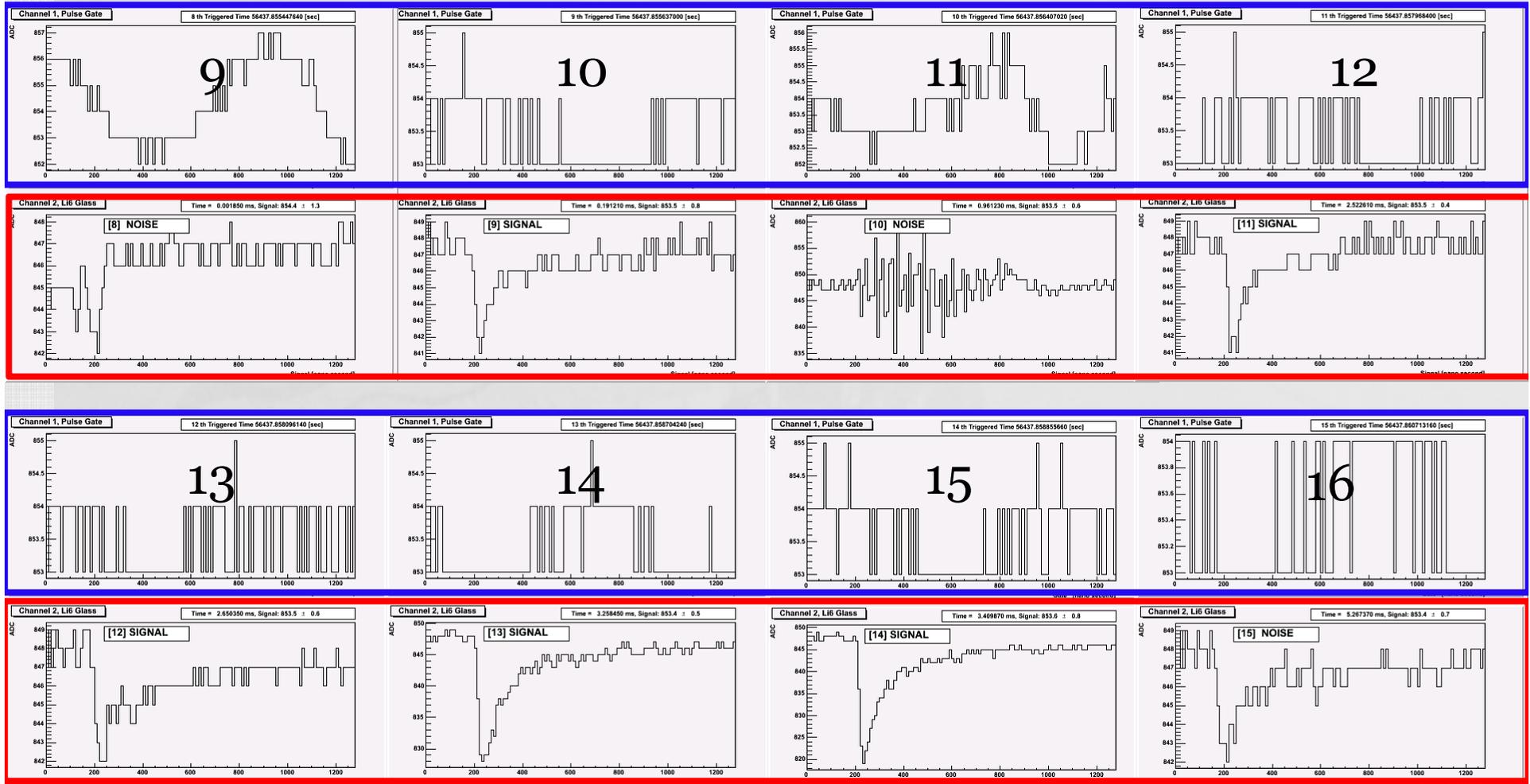
Recorded SIGNAL channel



4. Flash ADC TOF DAQ (4)

Recorded GATE channel

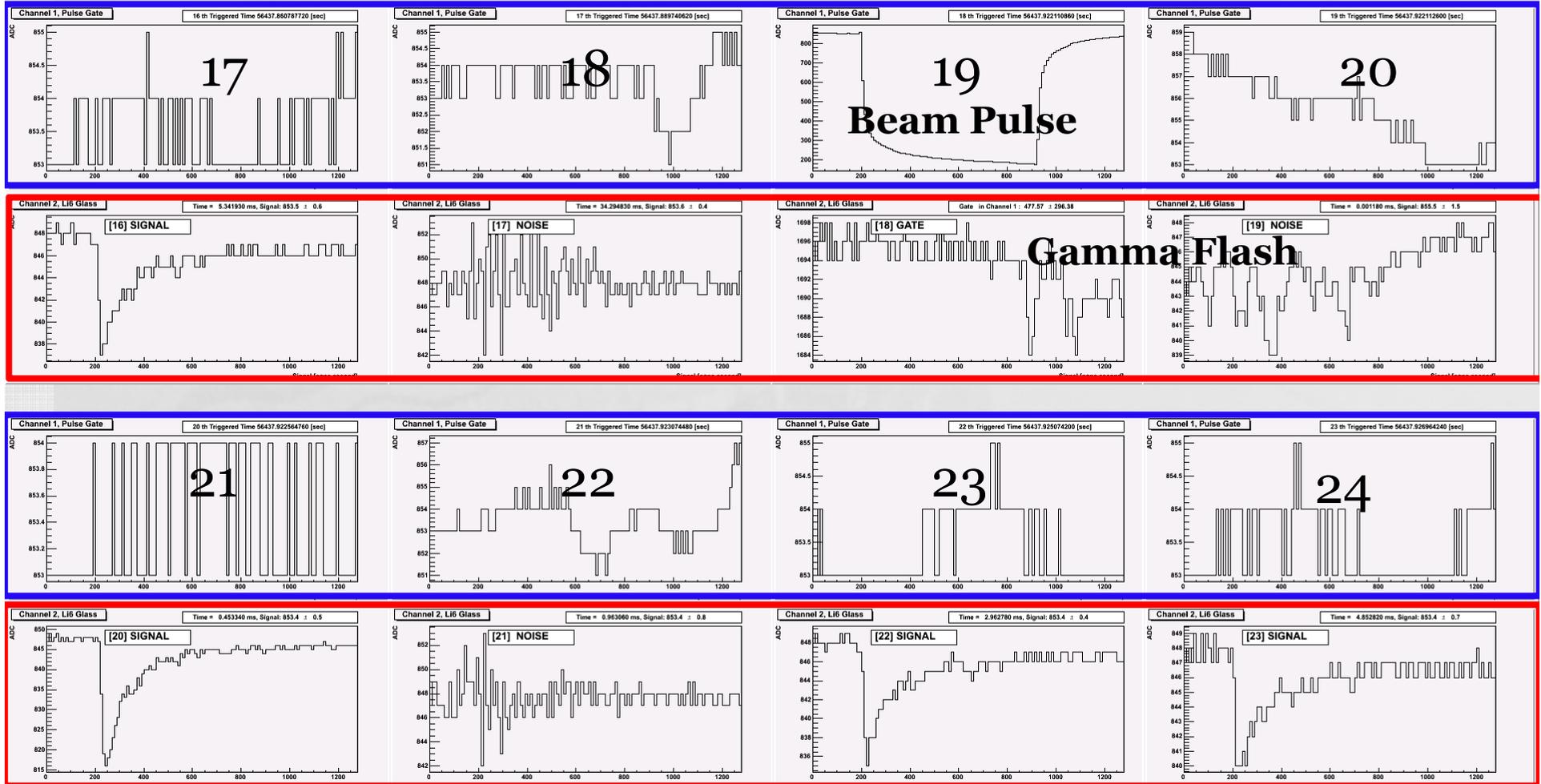
Recorded SIGNAL channel



4. Flash ADC TOF DAQ (5)

Recorded GATE channel

Recorded SIGNAL channel



5. Signal and Noise separation (1)

- 7 Parameters are studied to find best cuts to separate Signal and Noise
- 21 cases are studied and 4 are selected as a best conditions for a better S/N

1. timeTOF : triggered time[us] from GATE

2. Sigave: Average ADC values

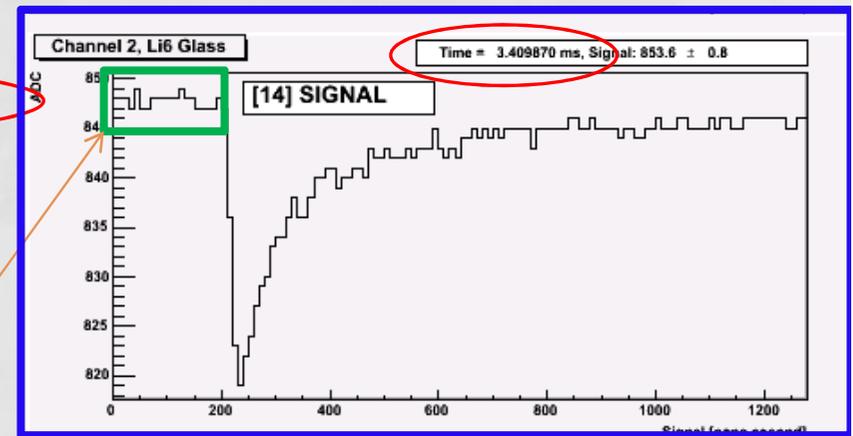
3. sigSE: Standard Error(SE) of Signals

4. **pedSE: SE of pedestal in first 20 bins**

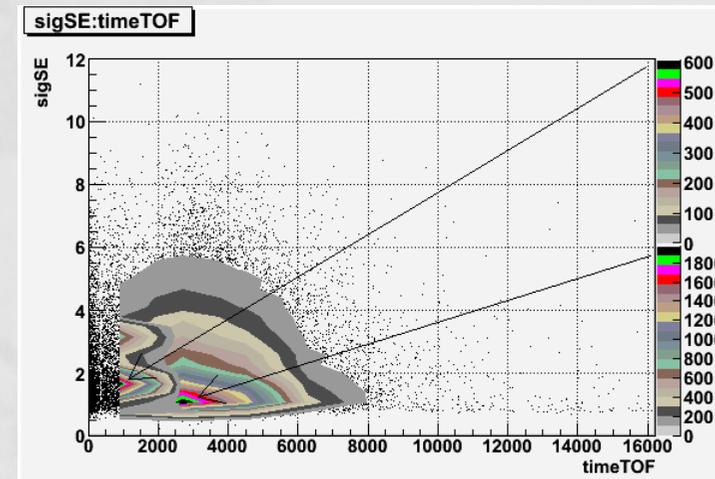
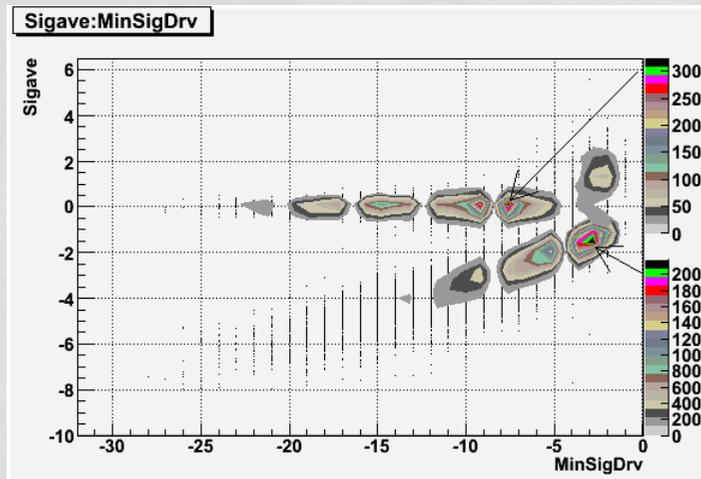
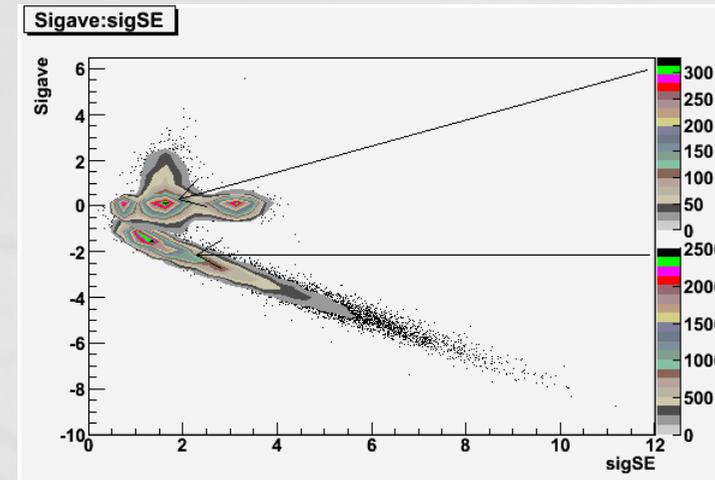
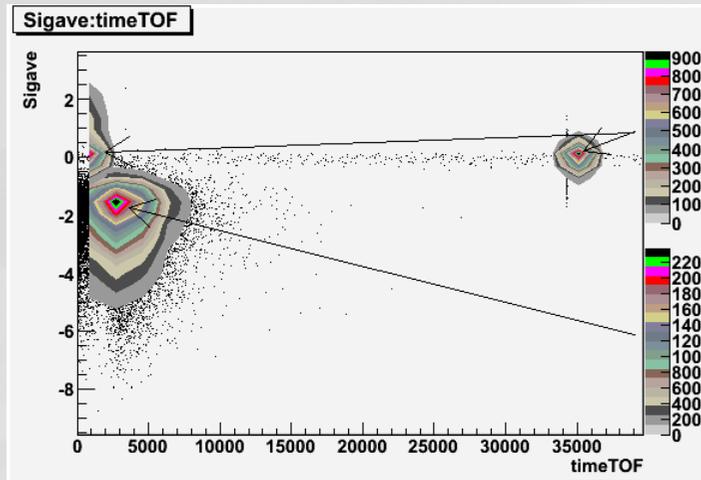
5. drvave: Average of first derivative of ADC

6. drvSE: SE of drvave

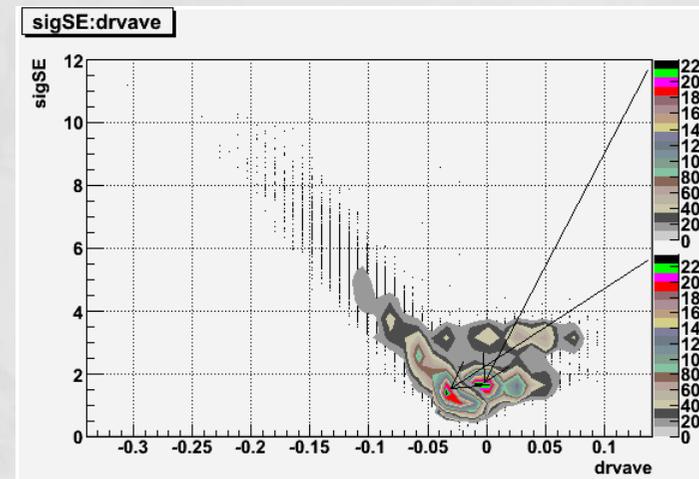
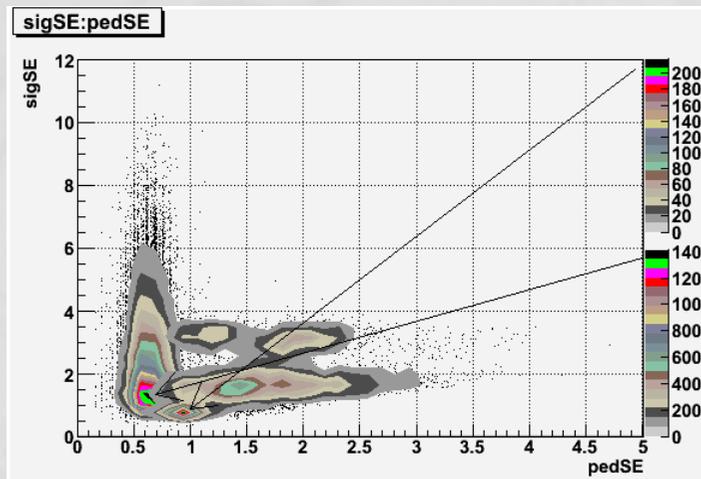
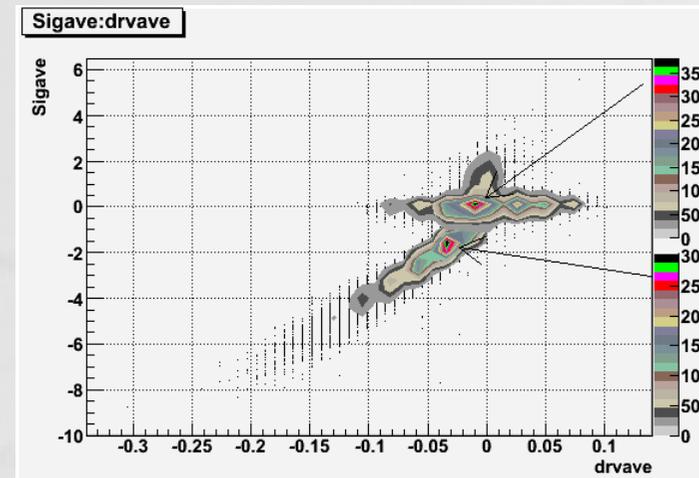
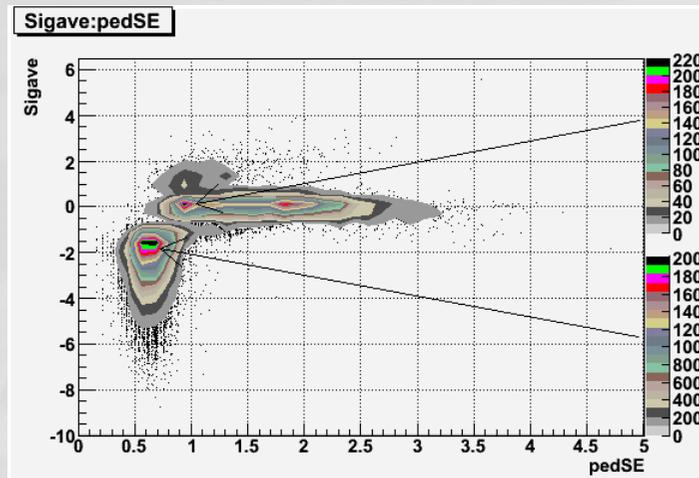
7. MinSigDrv: Minimum value of first derivative



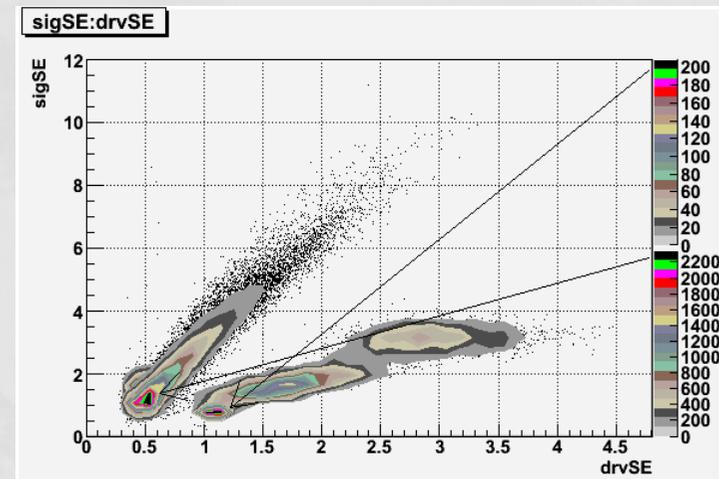
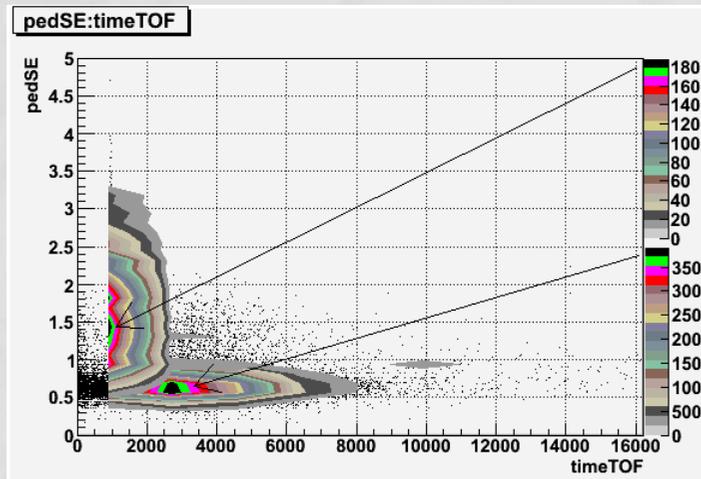
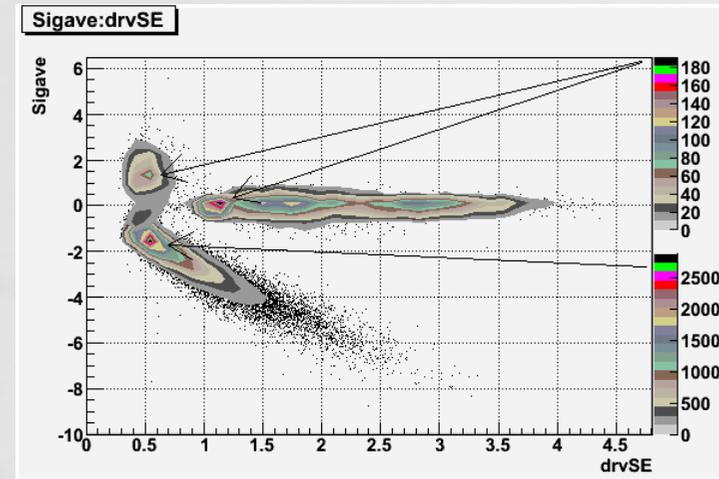
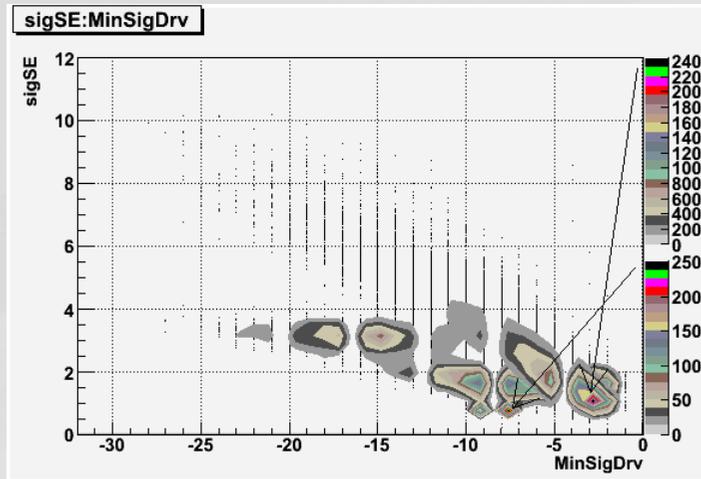
5. Signal and Noise separation (2)



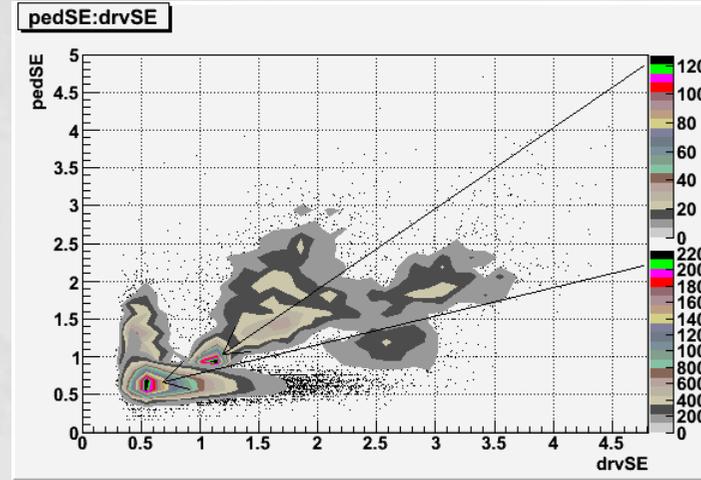
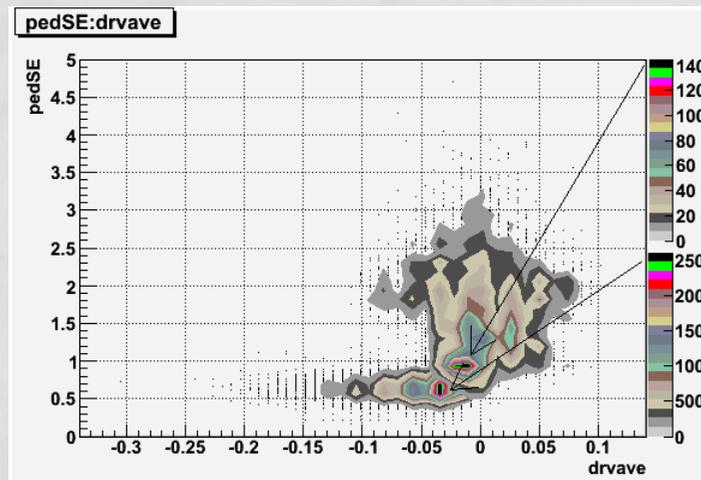
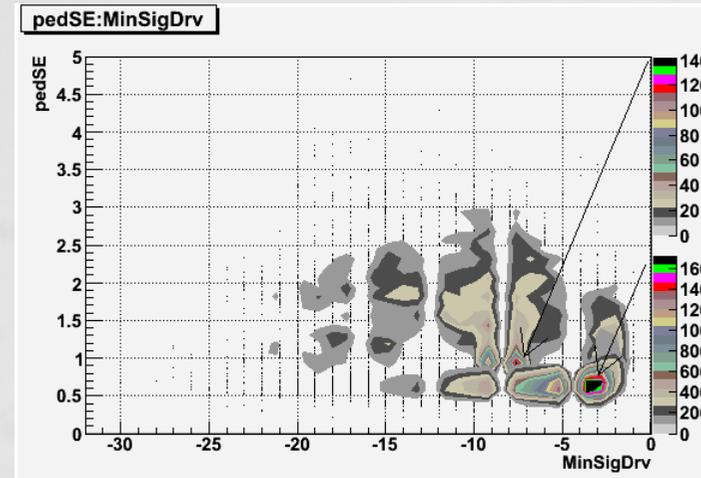
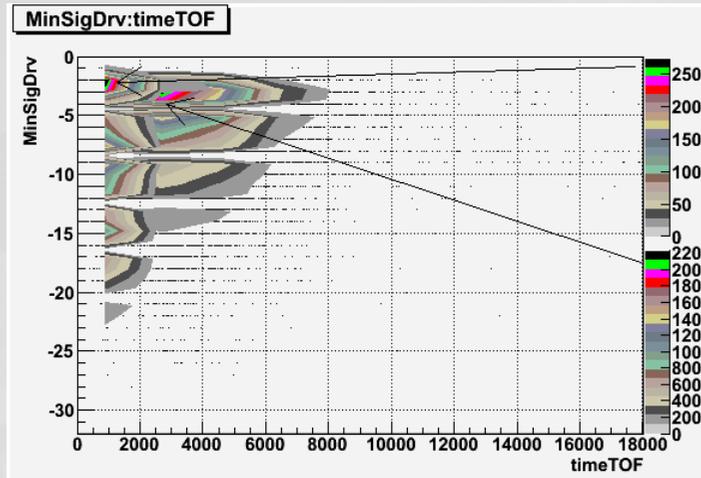
5. Signal and Noise separation (3)



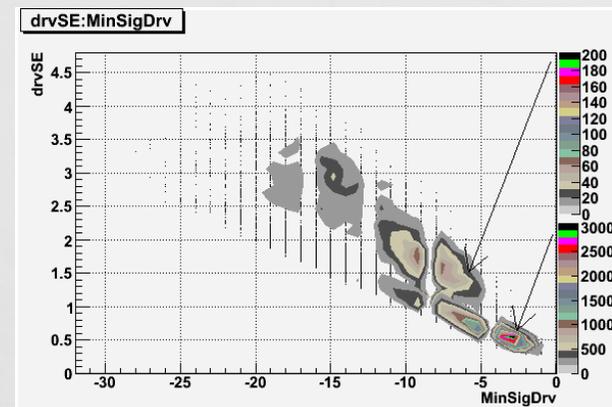
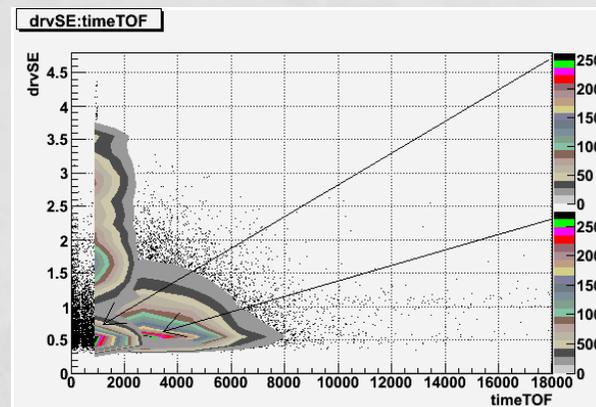
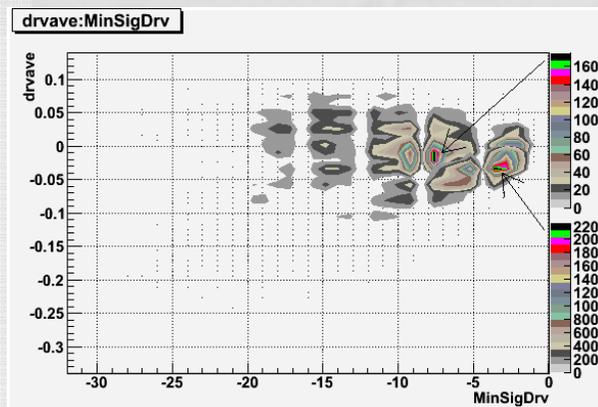
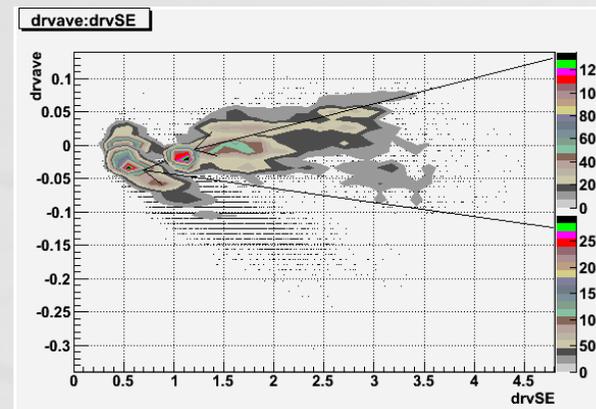
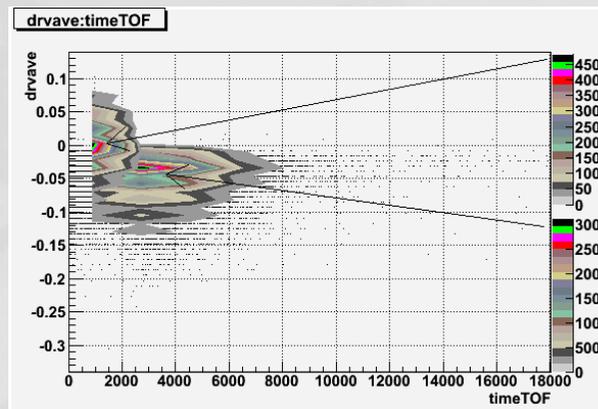
5. Signal and Noise separation (4)



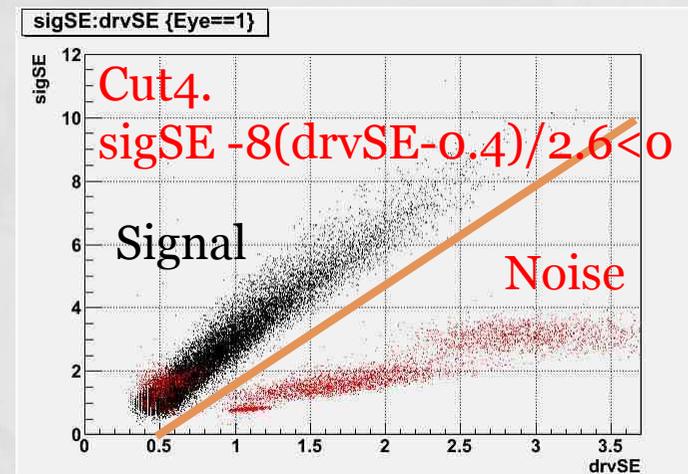
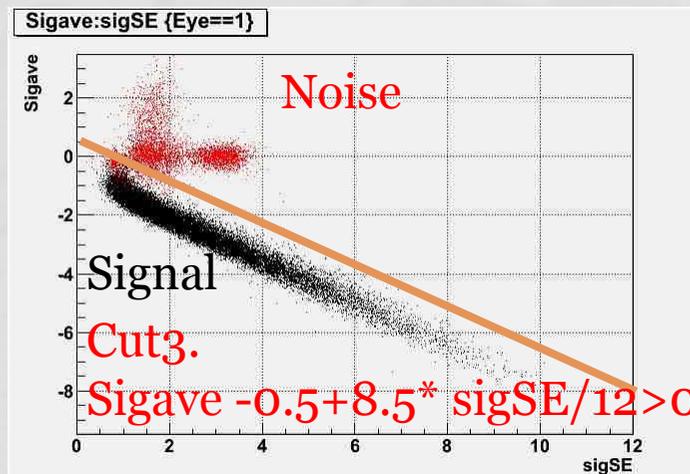
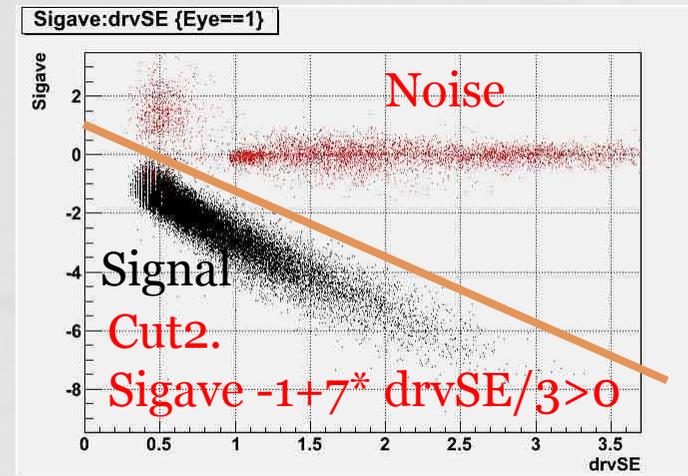
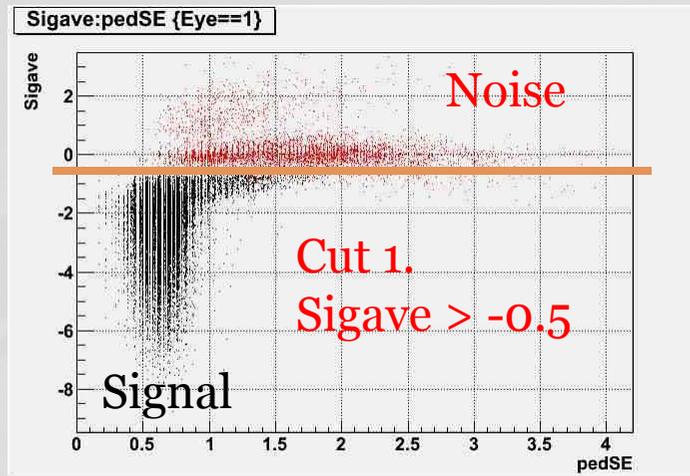
5. Signal and Noise separation (5)



5. Signal and Noise separation (6)



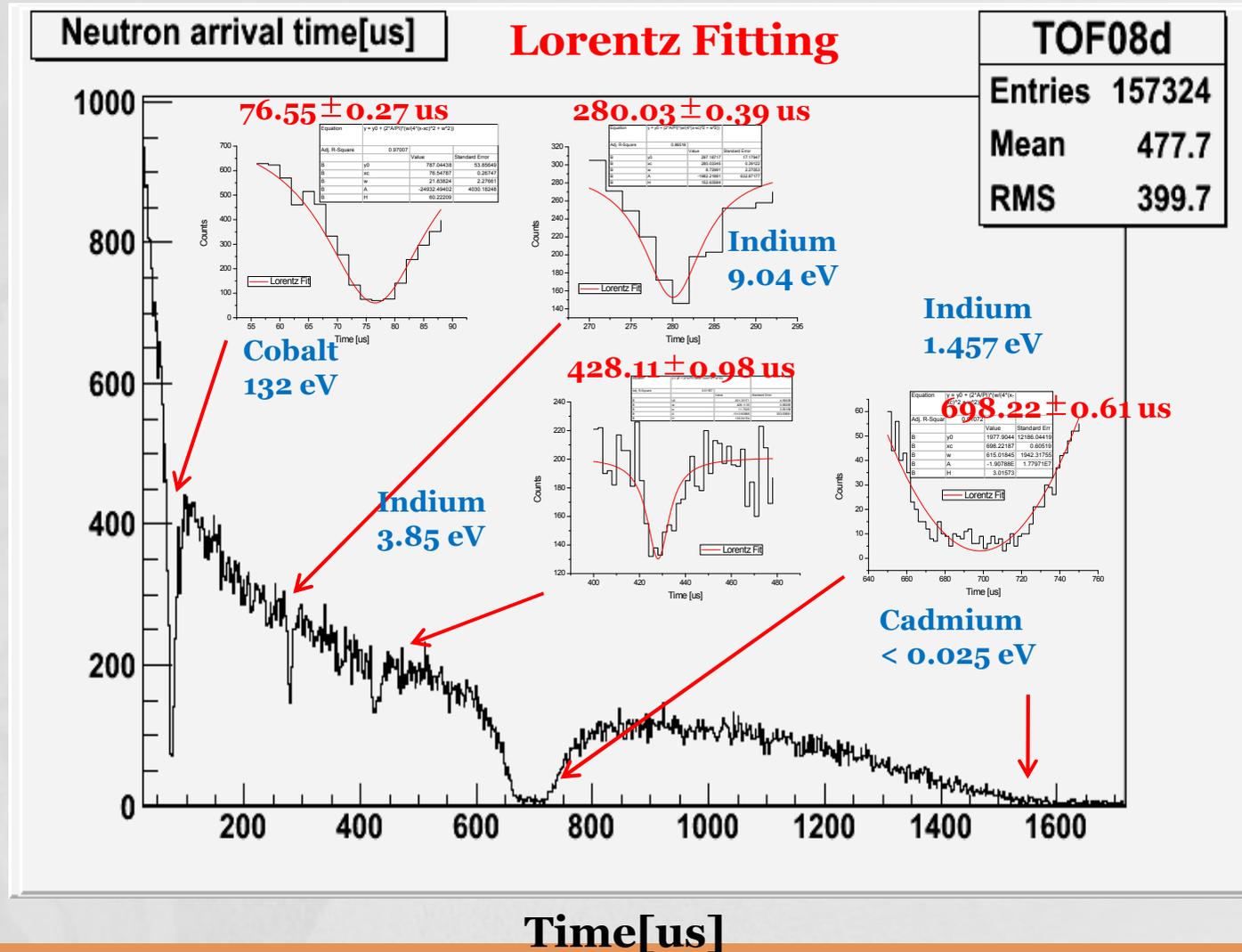
5. Signal and Noise separation (7)



4 cuts => S/N = 448.4, Signal Loss=1.19 %, Noise Rejection = 98.7 %

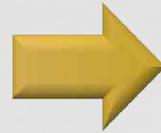
6. Neutron Energy Calibration

Using Notch Filter (Co, Cd, In)



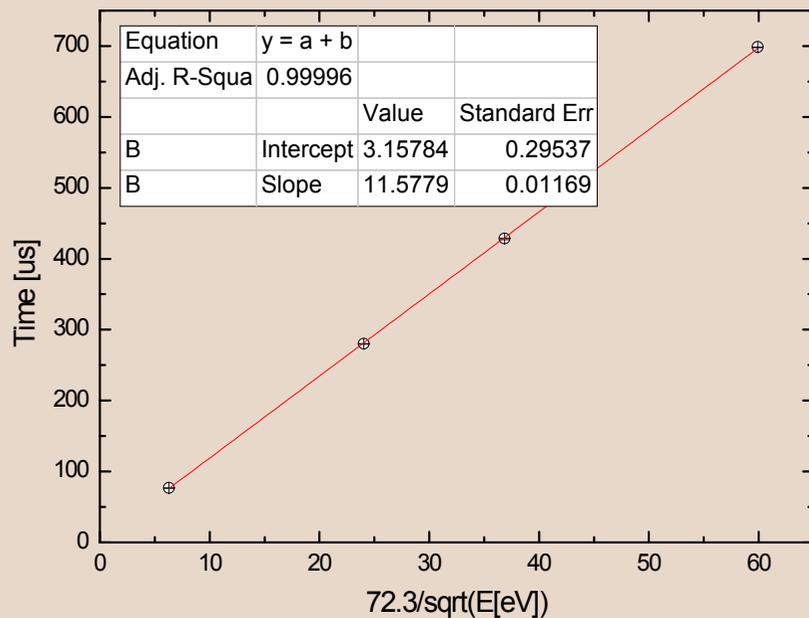
7. Flight Length and t_0 Fitting

$$E = \left(\frac{72.3 \times L[m]}{t - t_0[\mu s]} \right)^2 [eV]$$



$$t[\mu s] = \frac{72.3 \times L[m]}{\sqrt{E[eV]}} + t_0$$

Using a linear fitting of four energies from Notch Filter, L and t_0 are found

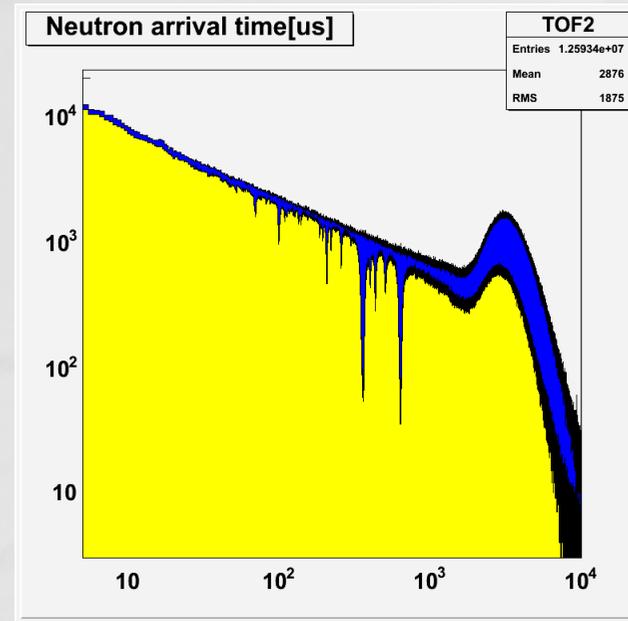
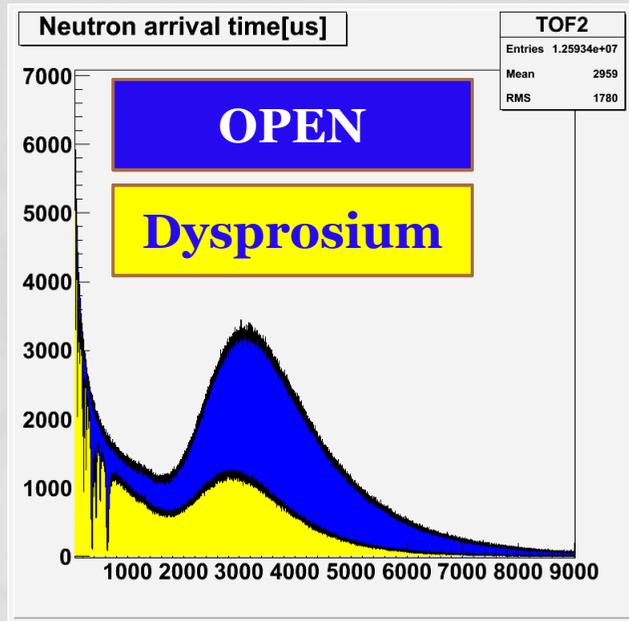


$$L = 11.578 \pm 0.012 [m]$$

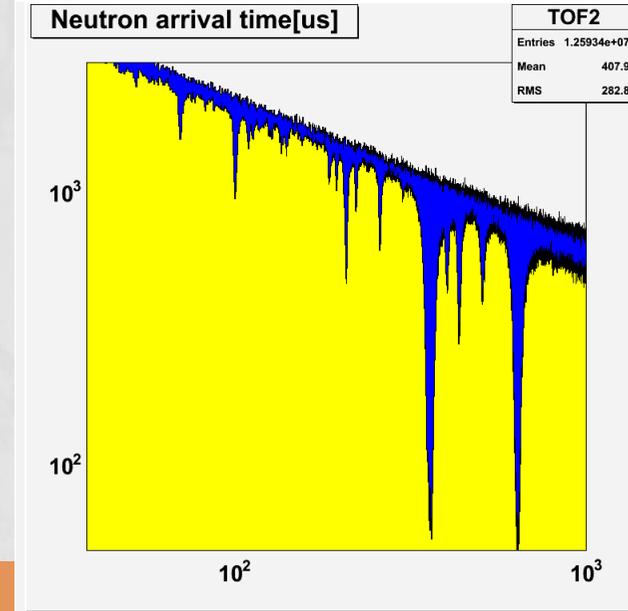
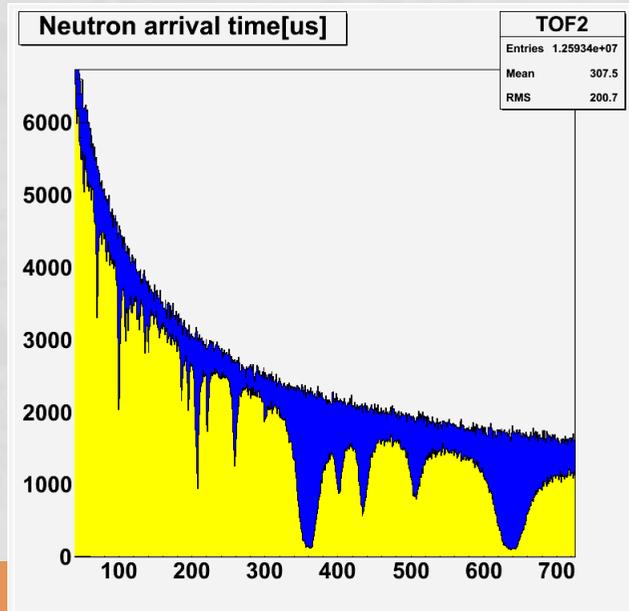
$$t_0 = 3.158 \pm 0.295 [\mu s]$$

$$E = \left(\frac{72.3 \times 11.578}{t[\mu s] - 3.158} \right)^2 [eV]$$

8. Dysprosium TOF Spectrum



Linear



Log

Neutron Resonance Analysis using SAMMY

Contents 2

1. SAMMY-8 Introduction
2. Pre_SAMMY Run
3. Input Data Preparation
4. SAMMY fitting

1. SAMMY Introduction (1)

OAK RIDGE NATIONAL LABORATORY

managed by
UT-BATTELLE, LLC
for the
U.S. DEPARTMENT OF ENERGY

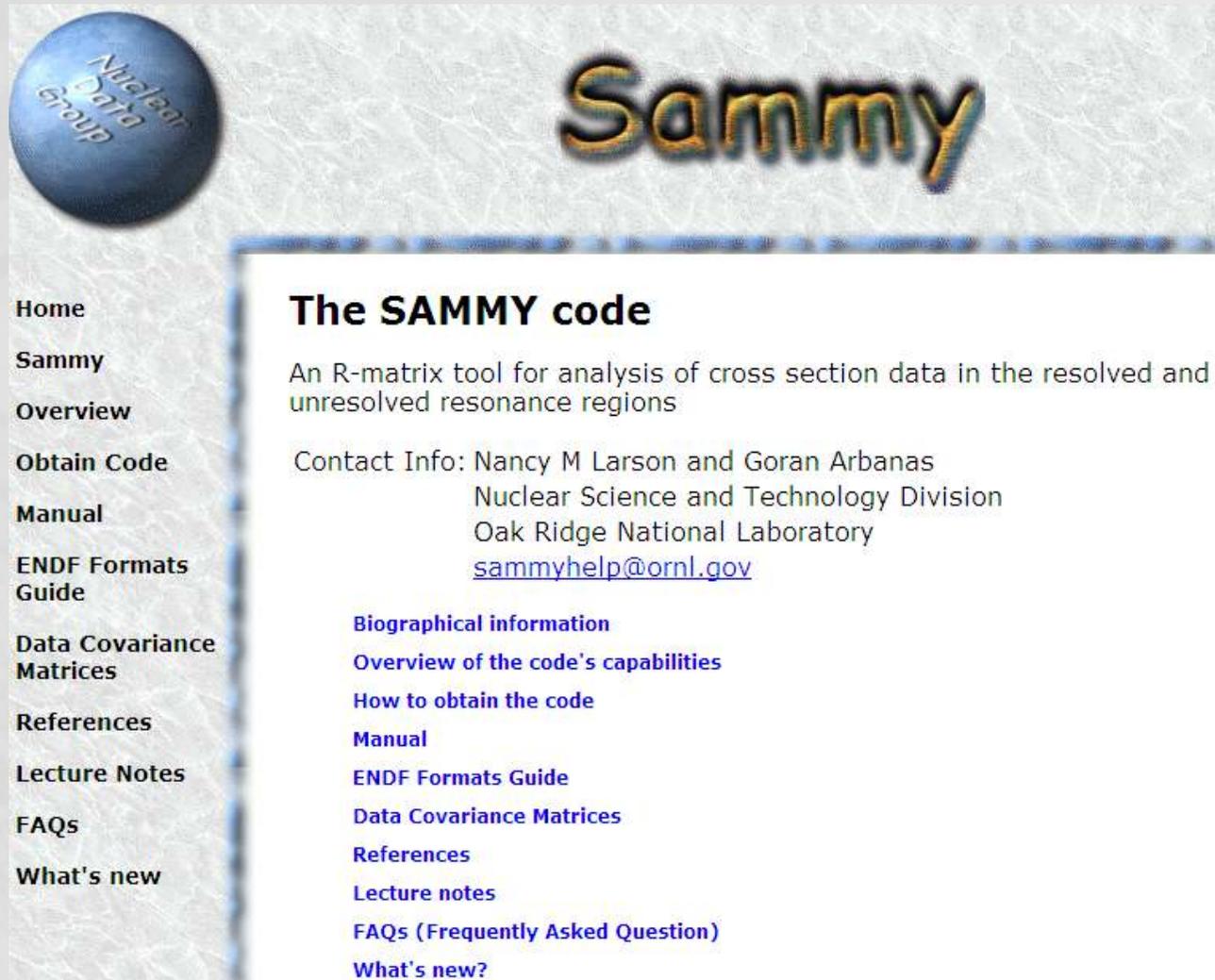
RSICC PERIPHERAL SCIENCE ROUTINE

SAMMY-8

**Code System for Multilevel R-Matrix Fits to Neutron
and Charged-Particle Cross-Section Data Using Bayes' Equations**

1. SAMMY Introduction (2)

http://www.ornl.gov/sci/nuclear_science_technology/nuclear_data/sammy/



Nuclear Data Group

Sammy

- Home
- Sammy
- Overview
- Obtain Code
- Manual
- ENDF Formats Guide
- Data Covariance Matrices
- References
- Lecture Notes
- FAQs
- What's new

The SAMMY code

An R-matrix tool for analysis of cross section data in the resolved and unresolved resonance regions

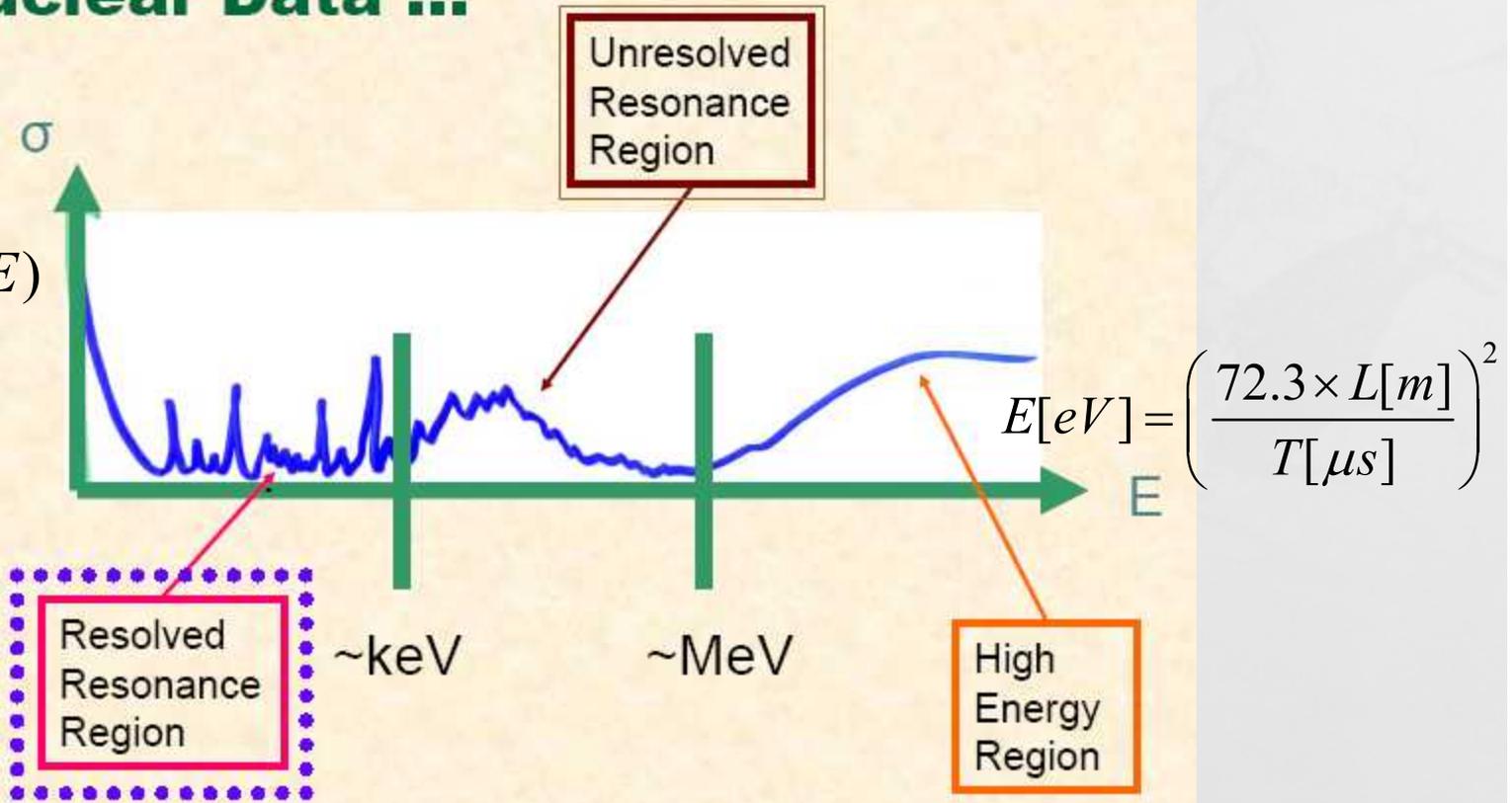
Contact Info: Nancy M Larson and Goran Arbanas
Nuclear Science and Technology Division
Oak Ridge National Laboratory
sammyhelp@ornl.gov

- [Biographical information](#)
- [Overview of the code's capabilities](#)
- [How to obtain the code](#)
- [Manual](#)
- [ENDF Formats Guide](#)
- [Data Covariance Matrices](#)
- [References](#)
- [Lecture notes](#)
- [FAQs \(Frequently Asked Question\)](#)
- [What's new?](#)

1. SAMMY Introduction (3)

$$\sigma(E) = \frac{1}{N} \ln T(E)$$

Nuclear Data ...



- Purpose: analyze time-of-flight cross-section data in the resonance region

1. SAMMY Introduction (4)

- *Types of data:*

- total, elastic, capture, fission, inelastic, other reactions
 - Coulomb or not
- angular distributions for elastic cross sections
- certain types of integral data

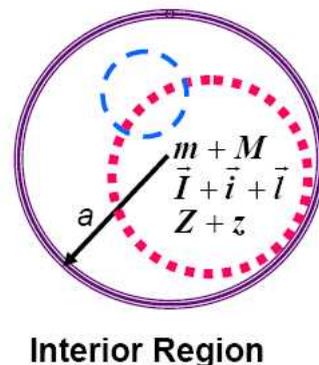
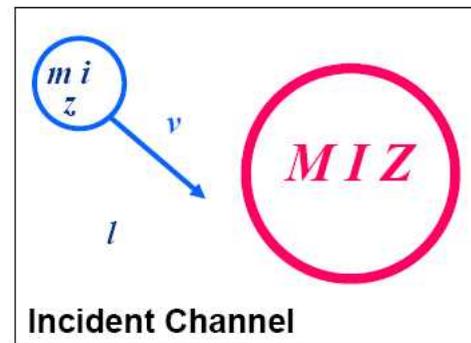
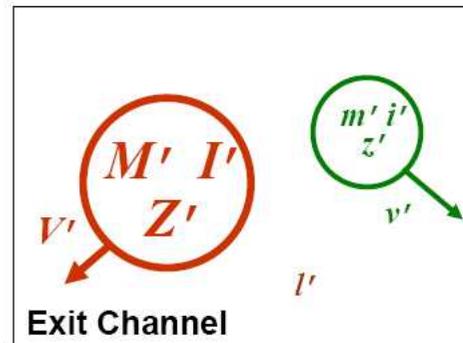


Figure II.1. Schematic of entrance and exit channels as used in scattering theory. For the interior region (with separation distance $r < a$), no assumptions are made about the nature of the interaction. In the figure, m , i , and z refer to the mass, spin, and charge of the incident particle while M , I and Z refer to the target particle. Orbital angular momentum is denoted by l and velocity by v . Primes are used for post-collision quantities.



1. SAMMY Introduction (5)

II.B.3.a. Single and multilevel Breit-Wigner cross sections

$$\sigma^{total} = \sigma^{elastic} + \sigma^{fission} + \sigma^{capture}$$

$$\sigma^{elastic} = \frac{\pi}{k^2} \sum_J g_J \sum_c \left\{ (1 - \cos 2\varphi) \left(2 - \sum_{\lambda} \Gamma_{\lambda c} \Gamma_{\lambda} / d_{\lambda} \right) + 2 \sin 2\varphi \sum_{\lambda} \Gamma_{\lambda c} (E - E_{\lambda}) / d_{\lambda} + \left(\sum_{\lambda} \Gamma_{\lambda c} (E - E_{\lambda}) / d_{\lambda} \right)^2 + \left(\sum_{\lambda} \Gamma_{\lambda c} \Gamma_{\lambda} / 2d_{\lambda} \right)^2 \right\}$$

$$\sigma^{fission} = \frac{\pi}{k^2} \sum_J g_J \sum_c \sum_{c'} \sum_{\lambda} \frac{\Gamma_{\lambda c} \Gamma_{\lambda c'}}{d_{\lambda}}$$

$$\sigma^{capture} = \frac{\pi}{k^2} \sum_J g_J \sum_c \sum_{\lambda} \frac{\Gamma_{\lambda c} \bar{\Gamma}_{\lambda \gamma}}{d_{\lambda}}$$

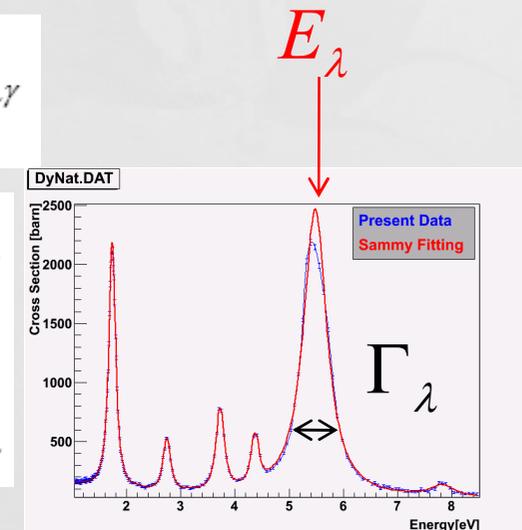
$$d_{\lambda} = (E - E_{\lambda})^2 + (\Gamma_{\lambda} / 2)^2$$

$$\Gamma_{\lambda} = \sum_c \Gamma_{\lambda c} + \bar{\Gamma}_{\lambda \gamma}$$

$$\Gamma_{\lambda c}^{neutron} = 2\gamma_{\lambda c}^2 P_c$$

$$\Gamma_{\lambda c}^{fission} = 2\gamma_{\lambda c}^2$$

$$\bar{\Gamma}_{\lambda \gamma} = 2\bar{\gamma}_{\lambda \gamma}^2$$



SAMMY will calculate Resonance Energy, Gamma-Neutron, Gamma-Gamma, etc.

Resonance Parameters Deduced by SAMMY

- SAMMY is a multilevel R-matrix code for fitting neutron time-of-flight cross-section data using Bayes' method.
- The resonance analysis with SAMMY code provides complete information on the covariance for the resonance's parameters
- The main feature of SAMMY code is analysis of neutron resonance parameters based on neutron transmission and capture experimental data

R-matrix formula

$$R_{cc'} = \sum_{\lambda} \frac{\gamma_{\lambda c} \gamma_{\lambda c'}}{E_{\lambda} - E - i\gamma_{\lambda\gamma}^2}$$

c, c' : particle channel except for capture channel

γ : reduced-width amplitude

Γ : reaction width

}	$\Gamma_{\lambda c} = 2P_1\gamma_{\lambda c}^2$	for neutron channel
	$\Gamma_{\lambda c} = 2\gamma_{\lambda c}^2$	for fission channel
	$\Gamma_{\lambda c} = 2\alpha_{\lambda}^2$	for gamma channel

P_1 : penetrability (penetration factor)

2. Pre_SAMMY Run(1)

- To make *.DAT, *.INP, *.PAR for Pre_Sammy run
- Pre_Dat file: (N, G)file_MT102 of ENDF_VII for Dy 156, 158, 160, 161, 162, 163, 164
- <http://www.nndc.bnl.gov>

Target Dy-156
56fe; fe-56; 26-fe-56; fe*

Reaction N,G
n, *; n,tot; n,g; n,f; n,inl; n,nu*

Quantity
sig; da; de; da/de; res; cov*

Library
 All Selected Reset

- ENDF/B-VII.0 (USA, 2006)
- JEFF-3.1 (Europe, 2005)
- JENDL-4.0 (Japan, 2010)
- CENDL-3.1 (China, 2009)
- RBEP/B-VI.0 (Russia, 2010)



Request #69445

ENDF Data Selection

Selected Unselected All

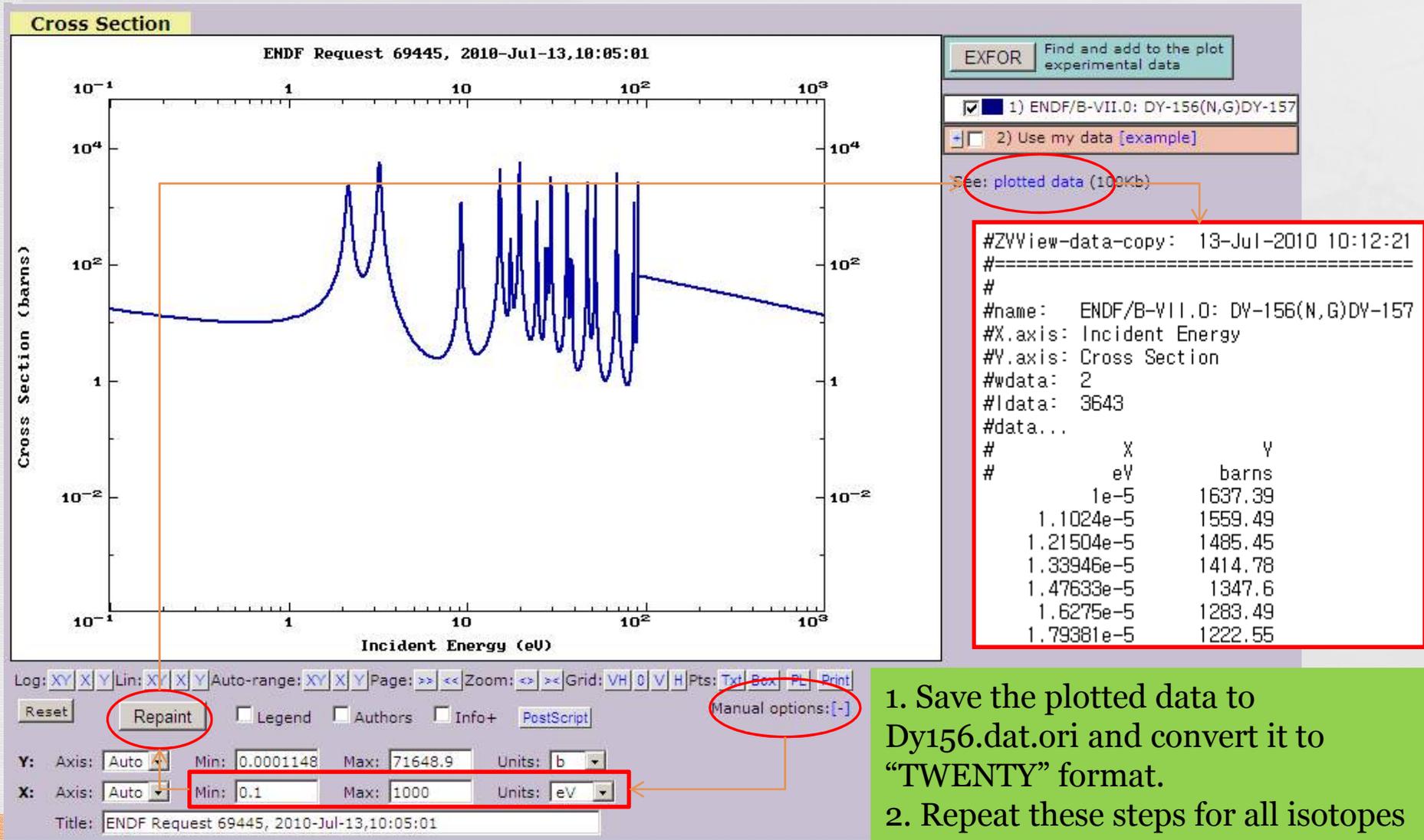
Plotting options: Quick plot (cross-sections only: σ)
 Universal plot ($\sigma \pm \Delta\sigma$, $d\sigma/d\Omega$, $d\sigma/dE$, $d^2\sigma/dE/d\Omega$) *beta version*

Sorted by: [Libraries] Reorder by: [Reactions] View: basic extended

1)	Info	Summary	MAT	66-DY-156	MAT=6625 NSUB=10(N)	20MeV	ENDF/B-VII.0	BNL,KAERI	Kim,Mughabghab,Herman,Oblozinsky	DIST-DEC06
MAT=6625 MF3 [SIG] Cross sections										
<input type="checkbox"/>	1	Summary	ENDF-6	Interpreted	Plot	DY-156(N,G)DY-157,SIG	MT102 QM=6.969e+6 QI=6.969e+6 LR=0			
MAT=6625 MF6 [DA/DE] Product energy-angle distributions										
<input type="checkbox"/>	2	Summary	ENDF-6	Interpreted	Plot	DY-156(N,G)DY-157,DA/DE	MT102 NK=1			
1: Lines:2-1654 Prod:G-0 ZAP=0 AWP=0 LIP=0 LAW=1										

2. Pre_SAMMY Run(2)

- Manual options:[-]→Energy -> eV 로 변환→ Repaint → See: plotted data



1. Save the plotted data to Dy156.dat.ori and convert it to "TWENTY" format.
2. Repeat these steps for all isotopes

2. Pre_SAMMY Run(3)

- *.PAR → **Input for Variable Parameters**

<http://www-nds.iaea.org/exfor/endl.htm>

Standard Request Examples: [1](#) [2](#) [3](#) [4](#) [5](#) [6](#) [7](#) Go to: [Advanced Request](#); [ENDF-Explorer](#)

Parameters:

Target DY-161 *
Reaction N.RES *
Quantity *

[More Parameters...](#)

Libraries: All Selected(1)

<input checked="" type="checkbox"/> Major Libraries	<input type="checkbox"/> Special Libraries
<input checked="" type="checkbox"/> 1) ENDF/B-VII.0 (USA,2006)	<input type="checkbox"/> Archival
<input type="checkbox"/> 2) JEFF-3.1.1 (Europe,2005-2009)	<input type="checkbox"/> Derived
<input type="checkbox"/> 3) JENDL-4.0 (Japan,2010)	
<input type="checkbox"/> 4) BROND-2.2 (Russia,1992)	
<input type="checkbox"/> 5) CENDL-3.1 (China,2009)	

Options:
Sort by: Reactions Evaluations

ENDF Data Selection

Selected Unselected All

Sorted by: [Reactions] Reorder by: [Libraries] View: basic extended

1) DY-161(N,RES),RES MT=151 MF=2 NSUB=10
MF2: [RES] Resonance parameters MT151: [N,RES] Resonance parameters that can be used to calculate cross sections at different t

1 [ENDF-6](#) [Interpreted](#) ENDF/B-VII.0 E=20MeV Lab=BNL,KAERI Date=DIST-DEC06 Kim,Herman,Oh,Oblozinsky

Output Data

Format	Data (Size)
ENDF	Text (59Kb) ZIP (12Kb)

2. Pre_SAMMY Run(4)

○ *.PAR → Input for Variable Parameters

```
[Ri03]
RIPL-2: Reference Input Parameter Library
IAEA, Vienna 2005, see
http://www-nds.iaea.org/RIPL-2/

*****
          1      451      459      16640 1451 232
          2      151      504      16640 1451 233
0.000000+0 0.000000+0      0      0      0      06640 1 099999
0.000000+0 0.000000+0      0      0      0      06640 0 0 0
6.616100+4 1.595440+2      0      0      1      06640 2151 1
6.616100+4 1.000000+0      0      0      2      06640 2151 2
1.000000-5 9.963000+2      1      2      0      06640 2151 3
2.500000+0 7.469997-1      0      0      1      06640 2151 4
1.595446+2 0.000000+0      0      0      1524      2546640 2151 5
-1.890000+0 3.000000+0 1.176400-1 1.084000-2 1.068000-1 0.000000+06640 2151 6
2.710000+0 3.000000+0 1.195614-1 5.614285-4 1.190000-1 0.000000+06640 2151 7
3.680000+0 2.000000+0 1.261360-1 2.136000-3 1.240000-1 0.000000+06640 2151 8
4.330000+0 2.000000+0 8.137999-2 1.380000-3 8.000000-2 0.000000+06640 2151 9

          ●          ●          ●
          ●          ●          ●

1.125000+4 4.318060+0 0.000000+0 7.729327-4 1.068000-1 0.000000+06640 2151 497
1.250000+4 4.310188+0 0.000000+0 7.715236-4 1.068000-1 0.000000+06640 2151 498
1.375000+4 4.302328+0 0.000000+0 7.701166-4 1.068000-1 0.000000+06640 2151 499
1.500000+4 4.294486+0 0.000000+0 7.687130-4 1.068000-1 0.000000+06640 2151 500
1.750000+4 4.278844+0 0.000000+0 7.659130-4 1.068000-1 0.000000+06640 2151 501
2.000000+4 4.263261+0 0.000000+0 7.631237-4 1.068000-1 0.000000+06640 2151 502
2.250000+4 4.247738+0 0.000000+0 7.603450-4 1.068000-1 0.000000+06640 2151 503
2.581200+4 4.227266+0 0.000000+0 7.566806-4 1.068000-1 0.000000+06640 2151 504
0.000000+0 0.000000+0      0      0      0      06640 2 099999
0.000000+0 0.000000+0      0      0      0      06640 0 0 0
0.000000+0 0.000000+0      0      0      0      0 0 0 0
0.000000+0 0.000000+0      0      0      0      0 -1 0 0
```

1. Save the rectangular part as Dy156.ENDF to extract parameters using SAMMY.
2. Repeat these steps for all isotopes

2. Pre_SAMMY Run(5)

○ *.INP → Input for Pre-SAMMY Run

```
Dy156.inp (~₩바탕 화면₩2010-Kor...IWpre_sammy₩Dy_sammy₩Dy156) - GVIM
파일(F) 편집(E) 도구(T) 문법(S) 버퍼(B) 창(W) 도움말(H)
Dy156
Dy156 155.924 0.00001 10000.0 5
PRINT THEORETICAL VALUES
USE NEW SPIN GROUP FORMAT
DO NOT SUPPRESS ANY INTERMEDIATE RESULTS
PRINT REDUCED WIDTHS
DO NOT SOLVE BAYES EQUATIONS
TWENTY
GENERATE PLOT FILE AUTOMATICAL
INPUT IS ENDF/B FILE 2
293.0 25.5960 0.000000 0.0000000 .000 0.0 0.000
6.5200 1.6485E-2
capture
```

```
0.samprerun.sh (~₩바탕 화면₩201...earData
파일(F) 편집(E) 도구(T) 문법(S) 버퍼(B) 창(W)
#!/bin/bash
rm -rf *SAM*
PWD=`pwd`
BASENAME=`basename $PWD`
sammy << EOF
$BASENAME.inp
$BASENAME.endf
$BASENAME.dat.ori_TWENTY.CNU $1 $2
EOF
if [ -f SAMMY.PLT ]; then
samplt << EOF
SAMMY.PLT
N
EOF
root -b -q 'SAMROOT.C("$BASENAME")'
fi
```

1. Only change name and atomic weight from Dy156.inp for all isotopes
2. Repeat these steps for all isotopes

Pre-SAMMY Run

[sammy@linux Dy156] o.samprerun.sh 0.0001 1000.0

2. Pre_SAMMY Run(6)

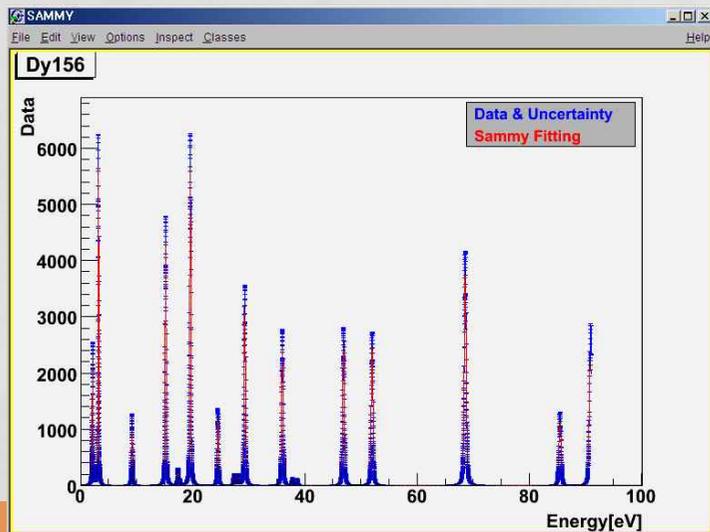
```

SAMNDF.INP (~₩바탕 화면₩2010-Ko...₩pre_sammy₩Dy_sammy₩Dy156) - GVIM2
파일(F) 편집(E) 도구(T) 문법(S) 버퍼(B) 창(W) 도움말(H)
Dy156
Dy156 155.924466 0.00001 1.0000E+04 0 5 0 0 0 0
PRINT THEORETICAL VALUES
USE NEW SPIN GROUP FORMAT
DO NOT SUPPRESS ANY INTERMEDIATE RESULTS
PRINT REDUCED WIDTHS
DO NOT SOLVE BAYES EQUATIONS
TWENTY
GENERATE PLOT FILE AUTOMATICAL
MLBW FORMALISM IS WANTED
PUT QUANTUM NUMBERS INTO PARAMETER FILE
293.0 25.5960 0.000000 0.0000000 0.00000.0 0.000
7.500000 0.016485 0.00000
CAPTURE
1 1 0 0.5 1.0 0.0
1 1 0 0 0.5
    
```

- Result of Pre-SAMMY Run →
- SAMNDF.INP & PAR
- Repeat Pre-SAMMY Run for all isotopes
- Combine these result to make real DyNAT.PAR file

```

SAMNDF.PAR (~₩바탕 화면₩2010-Ko...₩pre_sammy₩Dy_sammy₩Dy156) - GVIM2
파일(F) 편집(E) 도구(T) 문법(S) 버퍼(B) 창(W) 도움말(H)
2.150000000 83.6000000 .200000000 0 0 0 1
3.210000000 83.6000000 .800000000 0 0 0 1
9.190000000 83.6000000 .600000000 0 0 0 1
15.200000000 83.6000000 4.600000000 0 0 0 1
17.400000000 83.6000000 .320000000 0 0 0 1
17.700000000 83.6000000 .100000000 0 0 0 1
19.600000000 83.6000000 9.000000000 0 0 0 1
24.500000000 83.6000000 2.400000000 0 0 0 1
27.400000000 83.6000000 .400000000 0 0 0 1
28.100000000 83.6000000 .400000000 0 0 0 1
29.300000000 83.6000000 8.700000000 0 0 0 1
36.000000000 83.6000000 9.000000000 0 0 0 1
37.800000000 83.6000000 .400000000 0 0 0 1
38.600000000 83.6000000 .400000000 0 0 0 1
46.900000000 83.6000000 14.0000000 0 0 0 1
52.000000000 83.6000000 16.0000000 0 0 0 1
68.600000000 83.6000000 54.0000000 0 0 0 1
85.500000000 83.6000000 15.0000000 0 0 0 1
90.900000000 83.6000000 42.0000000 0 0 0 1
    
```



3. Input Data Preparation (1)

- SAMMY Input Data: *.DAT, *.INP, *.PAR
- *.DAT → Modify **an experimental data** using one of three data formats
- *.INP → **Input for FIXED Parameters**
- *.PAR → **Input for Variable Parameters:**
Resonance Energy, Gamma-neutron,
Gamma-Gamma, Gamma-Fission

3. Input Data Preparation (2)

● Experimental Data Formats

1. “USE CSISRS FORMAT FOR data”
2. “USE TWENTY SIGNIFICANT digits for experimental data”
3. “USE ENDF/B ENERGIES and cross sections MAT=abcd”

```
if ( format == "CSISRS" ) {  
    // Make CSISRS format [ E1 Y1 YE1 ]  
    //                      [ E2 Y2 YE2 ]  
    //                      [ .. .. ]  
    //                      [ 11f 11f 11f ] without blank space  
    if ( (Energy > Emin) && ( Energy < Emax) ) {  
        FORMAT = Form(" %10.4e %10.4e %10.4e", Energy, Barn, BarnError);  
        out1 << FORMAT << endl;  
    }  
}  
else if ( format == "TWENTY" ) {  
  
    // Make TWENTY format [ E1 Y1 YE1 ]  
    //                      [ E2 Y2 YE2 ]  
    //                      [ .. .. ]  
    //                      [ 20f 20f 20f ] without blank space  
    if ( (Energy > Emin) && ( Energy < Emax) ) {  
        FORMAT = Form("%20.10f%20.10f%20.10f", Energy, Barn, BarnError);  
        out1 << FORMAT << endl;  
    }  
}
```

```
else { // format == "ENDF/B"  
    // Make old default format [ E1 Y1 YE1/Y1 E2 Y2 YE2/Y2 E3 Y3 YE3/Y3 ]  
    //                      [ E4 Y4 YE4/Y4 E5 Y5 YE5/Y5 E6 Y6 YE6/Y6 ]  
    //                      [ ..... ]  
    //                      [ "14f" "14f7.5f" "14f" "14f7.5f" "14f" "14f7.5f" ]  
    if ( (Energy > Emin) && ( Energy < Emax) ) {  
        FORMAT = Form(" %14.8e %14.8e%7.5f", Energy, Barn, BarnError/Barn);  
        out1 << FORMAT;  
        j++;  
        if ( (j%3) == 0 ) { out1 << endl; }  
    }  
}
```



0.1035080180	0.4333768189	0.0046054958
0.1037130877	0.4286459982	0.0046045193
0.1039187759	0.4333532155	0.0046566990
0.1041250527	0.4287957549	0.0046119601

3. Input Data Preparation (3)

*.INP → Input for FIXED parameters

DyNAT.INP

Table VI A.1. Format of the INPut File

C:L	P,T	Variable name	Meaning (units)	Notes
1:1	1-80, A	TITLE		
2:1	1-10, A	ELMNT	Sample element's name	
	11-20, F	AW	Atomic weight (amu)	
	21-30, F	EMIN	Minimum energy for this data set (eV)	These values for EMIN and EMAX will be ignored if values are given in the interactive input (see Section VI.E)
	31-40, F	EMAX	Maximum energy (eV)	
	41-45, I	NEPNTS	1. Number of points to be used in generating artificial energy grid (default = 10001) 2. Maximum number of points to be analyzed at one time (default = 500). Use of this option is discouraged.	1. See Section V.A for a discussion of the artificial energy grid. 2. NEPNTS is the number of data points to be included in each region when "DIVIDE DATA INTO REGIONS" is specified in card set 3.

```

Dy EPITHERMAL TRANSMISSION - Dynat
Dynat 162.5 0.10000 6000. 5
PRINT THEORETICAL VALUES
PRINT ALL INPUT PARAMETERS
DO NOT SUPPRESS ANY INTERMEDIATE RESULTS
PRINT REDUCED WIDTHS
SOLVE BAYES EQUATIONS
TWENTY
EV
YIELD
FGM
GENERATE PLOT FILE AUTOMATICALLY
REICH-MOORE FORMALISM IS WANTED

293.0 11.51727 0.002260 0.1500000 -0.01800
0.01000 4
6.47906 256.000 30.10087 128.000 75.09176 64.000
1000.07062 32.000
7.500000 1.5824E-3 0.00100
TRANSMISSION
    
```

GROUPING part

***SOLVE → Data Fitting**
***DO NOT SOLVE →**
Theoretical Value Calculation
Using Parameter files

3. Input Data Preparation (4)

○ *.INP → Input for FIXED parameters

Continued of DyNat.INP

Look for Dysprosium of Wiki or Use Pre-SAMMY Run results

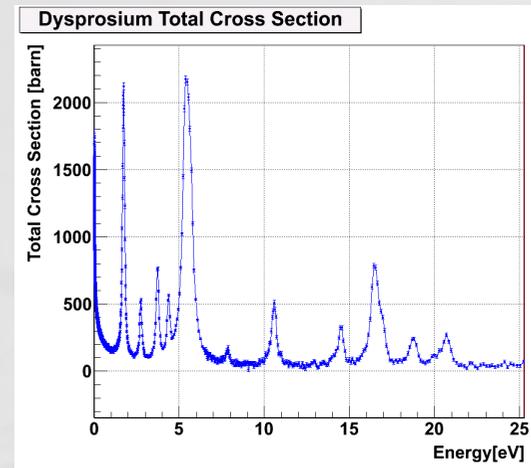
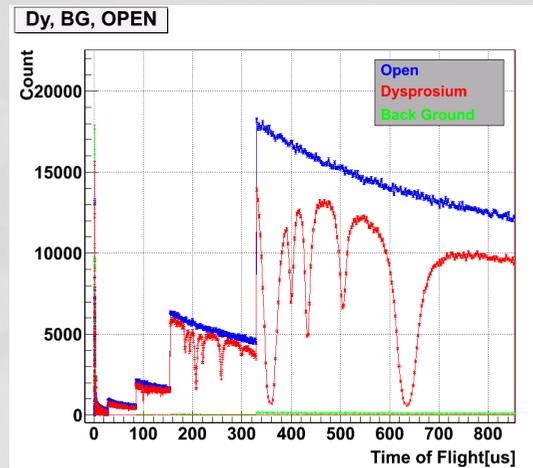
```

1 1 0 0.5 0.0006 0.0 Dy156
1 1 0 0 0.5
2 1 0 0.5 0.0010 0.0 Dy158
1 1 0 0 0.5
3 1 0 0.5 0.0234 0.0 Dy160
1 1 0 0 0.5
4 1 0 2.0 0.1891 2.5 Dy161
1 1 0 0 2.0
5 1 0 3.0 0.1891 2.5 Dy161
1 1 0 0 3.0
6 1 0 0.5 0.2551 0.0 Dy162
1 1 0 0 0.5
7 1 0 -0.5 0.2551 0.0 Dy162
1 1 0 1 0.5
8 1 0 -1.5 0.2551 0.0 Dy162
1 1 0 1 0.5
9 1 0 -2.0 0.2490 -2.5 Dy163
1 1 0 0 -2.0
10 1 0 -3.0 0.2490 -2.5 Dy163
1 1 0 0 -3.0
11 1 0 0.5 0.2818 0.0 Dy164
1 1 0 0 0.5
12 1 0 -0.5 0.2818 0.0 Dy164
1 1 0 1 0.5
13 1 0 -1.5 0.2818 0.0 Dy164
1 1 0 1 0.
    
```

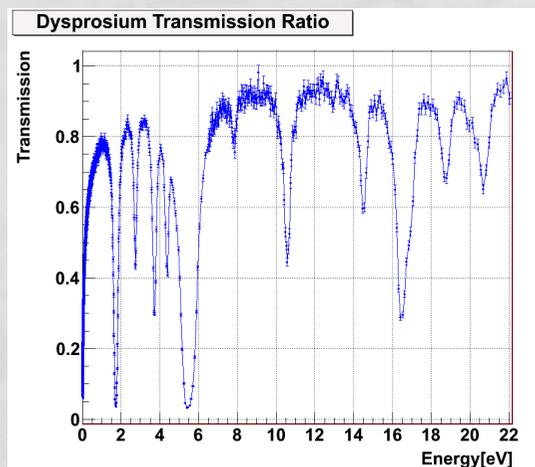
iso	NA	half-life	DM	DE (MeV)	DP
¹⁵⁴ Dy	syn	3.0×10 ⁸ y	α	2,947	¹⁵⁰ Gd
¹⁵⁶ Dy	0,06%	1×10 ¹⁸ y	α	?	¹⁵² Gd
¹⁵⁸ Dy	0,10%	¹⁵⁸ Dy is stable with 92 neutrons			
¹⁶⁰ Dy	2,34%	¹⁶⁰ Dy is stable with 94 neutrons			
¹⁶¹ Dy	18,91%	¹⁶¹ Dy is stable with 95 neutrons			
¹⁶² Dy	25,51%	¹⁶² Dy is stable with 96 neutrons			
¹⁶³ Dy	24,90%	¹⁶³ Dy is stable with 97 neutrons			
¹⁶⁴ Dy	28,18%	¹⁶⁴ Dy is stable with 98 neutrons			

^{155m} Dy	234,33(3) keV		6(1) μs	11/2-		
¹⁵⁶ Dy	66	90	155,924283(7)	STABLE [> 1E+18 a]	0+	0,00056(3)
¹⁵⁷ Dy	66	91	156,925466(7)	8,14(4) h	3/2-	
^{157m1} Dy	161,99(3) keV		1,3(2) μs	9/2+		
^{157m2} Dy	199,38(7) keV		21,6(16) ms	11/2-		
¹⁵⁸ Dy	66	92	157,924409(4)	STABLE	0+	0,00095(3)
¹⁵⁹ Dy	66	93	158,9257392(29)	144,4(2) d	3/2-	
^{159m} Dy	352,77(14) keV		122(3) μs	11/2-		
¹⁶⁰ Dy	66	94	159,9251975(27)	STABLE	0+	0,02329(18)
¹⁶¹ Dy	66	95	160,9269334(27)	STABLE	5/2+	0,18889(42)
¹⁶² Dy	66	96	161,9267984(27)	STABLE	0+	0,25475(36)
¹⁶³ Dy	66	97	162,9287312(27)	STABLE	5/2-	0,24896(42)
¹⁶⁴ Dy	66	98	163,9291748(27)	STABLE	0+	0,28260(54)

3. Sammy Fitting (1)



Grouping



Transmission

Cross Section and Resonance Parameters

Sammy Fitting using Input files

SAMMY Run

```
sammy DyNat.inp DyNat.par DyNat.dat 0.1 200
```

OUTPUT

```
SAM_ASCII.PLT → Fitted Cross Section  
SAM_ASCII_2.PLT → Fitted Transmission Rate  
SAMMY.PAR → Fitted variable parameters  
SAMMY.LPT → Total SAMMY Fitting Report
```

3. Sammy Fitting (2)

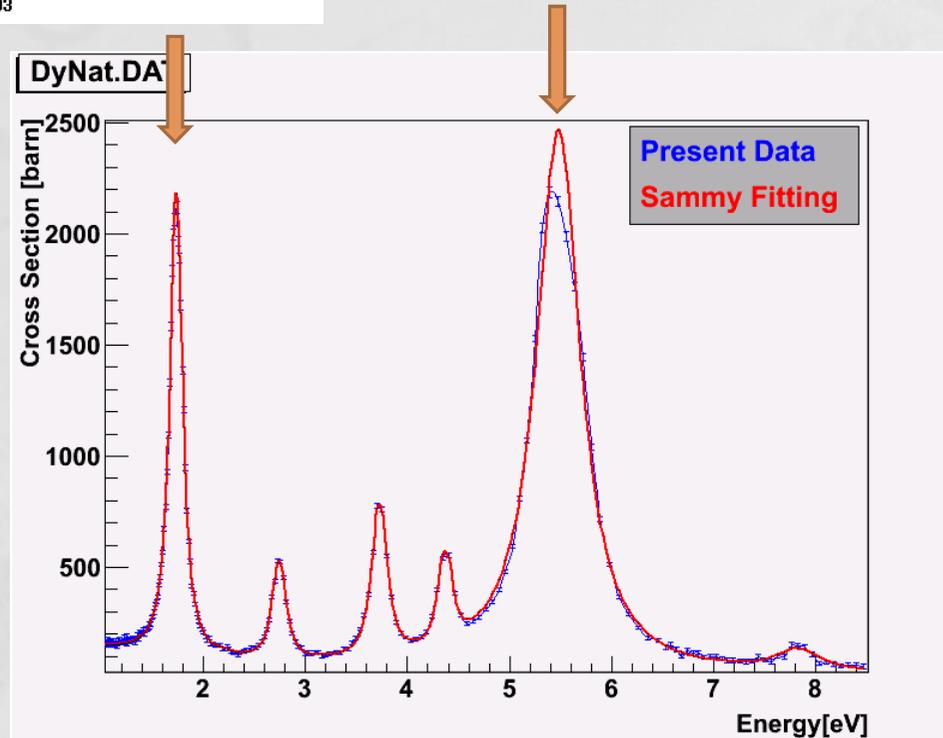
```

SPIN GROUP NUMBER 9 WITH SPIN= -2.0, ABUNDANCE= 0.2490, AND G= 0.4167
    effective radius = 7.4975E+00
    "true" radius = 7.4975E+00
ENERGY          GAMMA-          GAMMA-
                GAMMA          IncCha#6
                (MILLI-EU)      (MILLI-EU)
                (eU)          (eU)
1.73255E+00( 22) 1.2700E+02( 23) 1.8972E+00( 24)
6.00296E+00( 25) 1.0690E+02( 26) 3.4801E-02( 27)
Expected        1.2692E+02          5.1430E-01
Value           +/- 7.1022E-01 +/- 2.9020E-03
EU changed      1.2700E+02          1.8972E+00
parameters     +/- 7.1174E-01 +/- 5.7147E-03
    
```

```

SPIN GROUP NUMBER 6 WITH SPIN= 0.5, ABUNDANCE= 0.2551, AND G= 1.0000
    effective radius = 5.9000E+00
    "true" radius = 5.9000E+00
ENERGY          GAMMA-          GAMMA-
                GAMMA          IncCha#5
                (MILLI-EU)      (MILLI-EU)
                (eU)          (eU)
5.48424E+00( 19) 4.9728E+02( 20) 1.0262E+01( 21)
Expected        4.9728E+02          1.0262E+01
Value           +/- 3.1652E+00 +/- 3.2261E-02
EU changed      4.9728E+02          1.0262E+01
parameters     +/- 3.1652E+00 +/- 3.2261E-02
    
```

First
SAMMY
Fitting
Result



3. Sammy Fitting (3)

First RESULTS of SAMMY

SAM_ASCII.PLT → Fitted Cross Section

SAMMY.PAR → Fitted variable parameters

Energy	Data	Uncertainty	Th_initial	Th_final
1.3025376	181.30036	8.5137938	164.72168	164.65109
1.3117266	173.78704	8.4353505	166.51183	166.44314
1.3210131	179.57890	8.4119245	168.48514	168.41849
1.3303987	162.78007	8.4507571	170.66095	170.59646
1.3398849	180.73825	8.4653113	173.06178	172.99962
1.3494726	197.27698	8.4692367	175.71328	175.65362
1.3591636	177.34523	8.3897087	178.64514	178.58816
1.3689594	206.74253	8.4714438	181.89162	181.83752
1.3788614	196.01652	8.5047413	185.49237	185.44137
1.3888714	203.90112	8.5303637	189.49377	189.44611
1.3989906	201.16411	8.4983986	193.94981	193.90576

E_λ	Γ_γ	Γ_{c1}		
1944.300000	105.800000	240.000000	0 0 0	3
1994.300000	105.800000	250.000000	0 0 0	3
-1.89000000	106.800000	10.8400000	0 0 0	5
2.741176157	136.380489	.666679893	1 1 1	5
3.724688419	144.286082	2.16324484	1 1 1	4
4.368586984	113.451917	1.32007559	1 1 1	4
7.740000000	107.000000	.514285800	0 0 0	5
10.26000000	106.800000	.288000000	0 0 0	4
10.85000000	106.800000	.411428600	0 0 0	5
12.65000000	106.800000	.048000000	0 0 0	5

SAM_ASCII_2.PLT → Fitted Transmission Rate

SAMMY.LPT → Total SAMMY Fitting Report

Energy	Transmission	Uncertainty	Tr_initial	Tr_final
1.3025376	0.75092119	1.01012373E-02	0.77054626	0.77063234
1.3117266	0.75988853	1.01276832E-02	0.76836660	0.76845012
1.3210131	0.75296640	1.00075565E-02	0.76597106	0.76605186
1.3303987	0.77321935	1.03241764E-02	0.76333836	0.76341626
1.3398849	0.75158840	1.00526391E-02	0.76044388	0.76051868
1.3494726	0.73220289	9.79789530E-03	0.75725995	0.75733144
1.3591636	0.75562847	1.00164143E-02	0.75375487	0.75382283
1.3689594	0.72133386	9.65496800E-03	0.74989259	0.74995679
1.3788614	0.73366255	9.85858400E-03	0.74563198	0.74569215
1.3888714	0.72457951	9.76586430E-03	0.74092569	0.74098157
1.3989906	0.72771972	9.77143460E-03	0.73571963	0.73577091

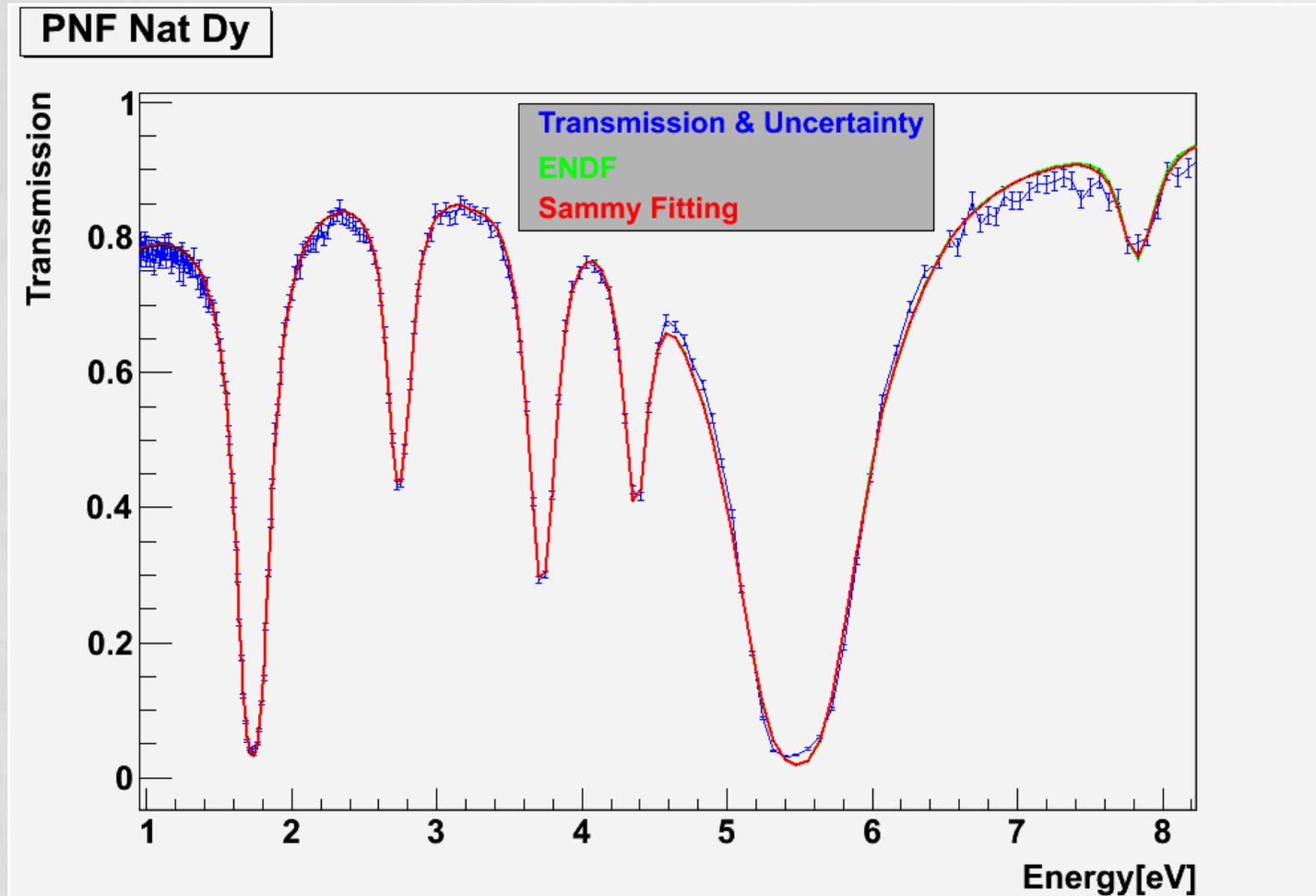
```

SAMMY.LPT (~\바탕 화면\W\ND2010\Wdy\Wsammy) - GVIM8
파일(F) 편집(E) 도구(T) 문법(S) 버퍼(B) 창(W) 도움말(H)
( 51)1.915779 0.583301 (104)3.194038 0.844324 (157)6.357799 0.729370
( 52)1.932188 0.619759 (105)3.229429 0.842182 (158)6.457529 0.771010
( 53)1.948809 0.650835 (106)3.265412 0.840388
### Array size used for SAMMY-INT is 14066 ###
### Estimated array size for SAMMY-IPQ is 14182 ###

CUSTOMARY CHI SQUARED = 1015.76
CUSTOMARY CHI SQUARED DIVIDED BY NDAT = 6.42883
### Array size used for SAMMY-IPQ is 12128 ###
### Estimated array size for SAMMY-FIN is 11596 ###
### Array size used for SAMMY-FIN is 8351 ###
Total time = 0.37 seconds

Normal finish to SAMMY
    
```

3. Sammy Fitting (4)



Second
SAMMY
Fitting
Result

3. Sammy Fitting (5)

Second RESULTS of SAMMY

SAM_ASCII.PLT → Fitted Cross Section

SAMMY.PAR → Fitted variable parameters

Energy	Data	Uncertainty	Th_initial	Th_final
1.3025376	181.30036	8.5137938	165.28435	165.16002
1.3117266	173.78704	8.4353505	167.07284	166.94572
1.3210131	179.57890	8.4119245	169.04455	168.91443
1.3303987	162.78007	8.4507571	171.21931	171.08598
1.3398849	180.73825	8.4653113	173.61978	173.48301
1.3494726	197.27698	8.4692367	176.27173	176.13126
1.3591636	177.34523	8.3897087	179.20501	179.06057
1.3689594	206.74253	8.4714438	182.45407	182.30536
1.3788614	196.01652	8.5047413	186.05882	185.90550
1.3888714	203.90112	8.5303637	190.06591	189.90761
1.3989906	201.16411	8.4983986	194.52969	194.36602

1944.300000	105.800000	240.000000		
1994.300000	105.800000	250.000000	0 0 0	3
-1.89000000	106.800000	10.84000000	0 0 0	5
2.740993036	137.546784	.670462372	1 1 1	5
3.724466930	143.806986	2.16238008	1 1 1	4
4.368726402	114.377050	1.33812149	1 1 1	4
7.829002600	212.322502	.838869818	1 1 1	5
10.04930701	119.397391	.362273576	1 1 1	4
11.02562238	120.660458	.345125992	1 1 1	5
12.84177451	105.542074	.054033458	1 1 1	5

$$E_{\lambda} \quad \Gamma_{\gamma} \quad \Gamma_{c1}$$

SAM_ASCII_2.PLT → Fitted Transmission Rate

SAMMY.LPT → Total SAMMY Fitting Report

Energy	Transmission	Uncertainty	Tr_initial	Tr_final
1.3025376	0.75092119	1.01012373E-02	0.76986050	0.77001197
1.3117266	0.75988853	1.01276832E-02	0.76768478	0.76783923
1.3210131	0.75296640	1.00075565E-02	0.76529332	0.76545091
1.3303987	0.77321935	1.03241764E-02	0.76266421	0.76282513
1.3398849	0.75158840	1.00526391E-02	0.75977272	0.75993718
1.3494726	0.73220289	9.79789530E-03	0.75659106	0.75675925
1.3591636	0.75562847	1.00164143E-02	0.75308739	0.75325953
1.3689594	0.72133386	9.65496800E-03	0.74922546	0.74940179
1.3788614	0.73366255	9.85858400E-03	0.74496393	0.74514469
1.3888714	0.72457951	9.76586430E-03	0.74025520	0.74044064
1.3989906	0.72771972	9.77143460E-03	0.73504483	0.73523523

```

(317)0.208697 0.549634 (634)0.678086 0.742931
### Array size used for SAMMY-INT is 34850 ###
### Estimated array size for SAMMY-NPU is 477421 ###

CUSTOMARY CHI SQUARED = 2316.83
CUSTOMARY CHI SQUARED DIVIDED BY NDATA = 2.43877
### Array size used for SAMMY-NPU is 930888 ###
### Estimated array size for SAMMY-FIN is 22844 ###
### Array size used for SAMMY-FIN is 8904 ###
Total time = 22.45 seconds

Normal finish to SAMMY
    
```

Development of Capture Cross Section Measurement System of Pohang Neutron Facility

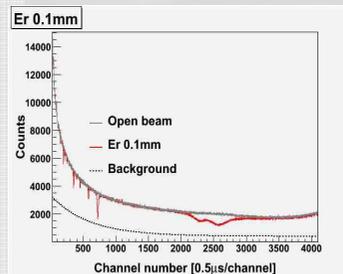
Contents 3

- Neutron Capture Cross-section
- BGO Crystal Study
- 4π BGO Gamma Detector
- BGO detector modules
- BGO Temperature Compensation
- DAQ system using FADC
- Current Status

Neutron Capture Cross-section

Total Cross-section

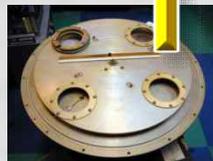
Capture Cross-section



Detector

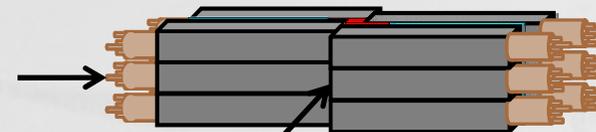
Neutron

Scattering,
Capture, etc



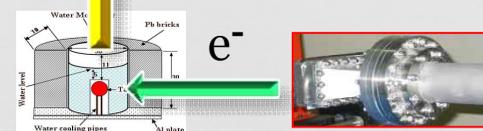
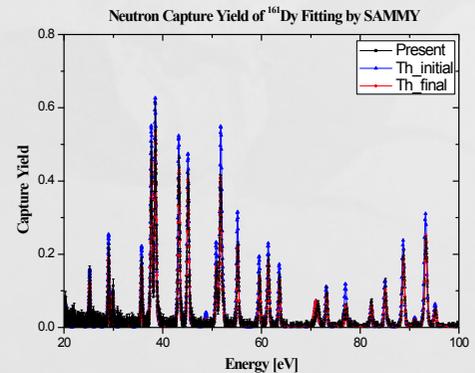
Sample
Changer

Neutron



Sample
In the middle

Neutron



Measurement with a total energy absorption detector by TOF method

Relation between neutron capture yield and cross sections:

$$Y(E_i) = (1 - e^{-N\sigma_t(E_i)t}) \frac{\sigma_c(E_i)}{f_c(E_i)\sigma_t(E_i)}$$

Y(E_i): neutron capture yield
N: atomic density of sample
t: thickness of sample
σ_t(E_i): neutron total cross section
σ_c(E_i): neutron capture cross section
f_c(E_i): correction function for neutron scattering and/or self-shielding in sample

If the thickness of sample is **thin** enough, capture cross section is

$$\sigma_c(E_i) = \frac{Y(E_i)}{Nt} f_c(E_i)$$

Neutron capture yield in sample:

$$C(E_i) = \varepsilon(E_i) Y(E_i) \phi(E_i)$$



$$Y(E_i) = \frac{C(E_i)}{\varepsilon(E_i) \phi(E_i)}$$

C_s(E_i): Counting rate for the sample by the detector system
ε_s(E_i): Gamma detection efficiency of the sample
φ(E_i): Neutron flux impinging on the sample as a function of neutron energy

Characteristics of Crystal

Scintillator	BGO	NaI (Tl)	CsI (Tl)
Wave length of max. Emission [nm]	480	410	560
Decay constant [ns]	300	230	1000
Refractive index	2.15	1.85	1.79
Scintillation efficiency [%]	8	100*	136.8 ~ 47.3
Specific Gravity	7.13	3.67	4.51
	Non-hygroscopic	Hygroscopic	Hygroscopic

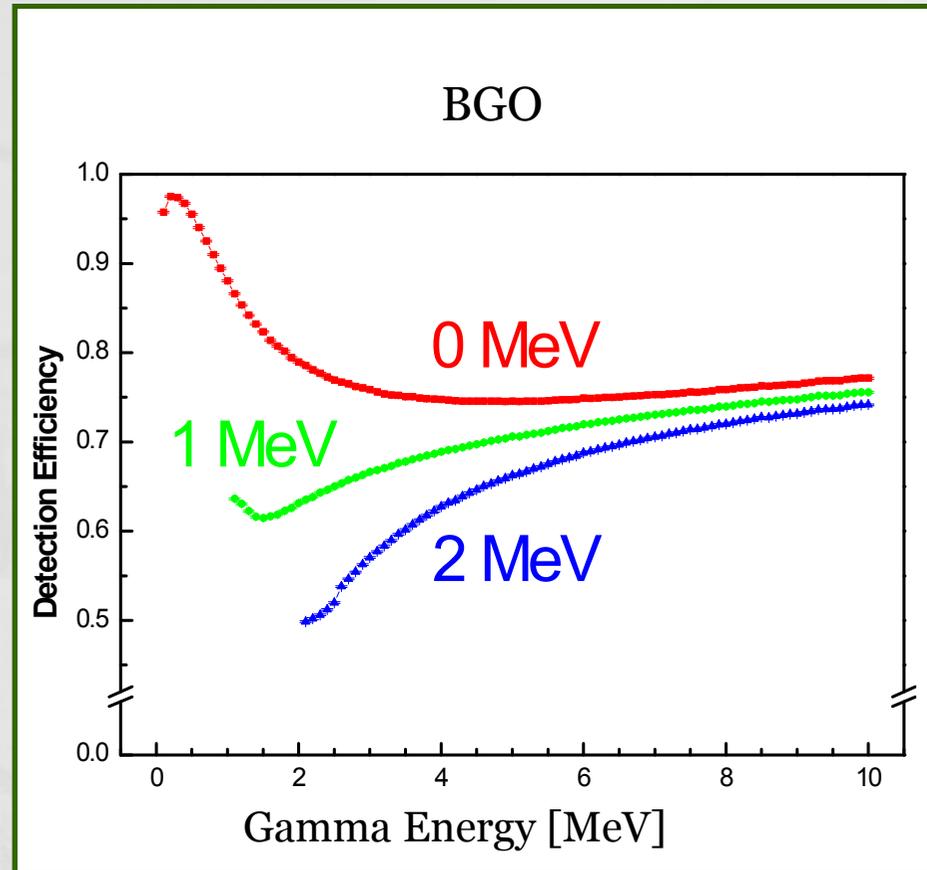
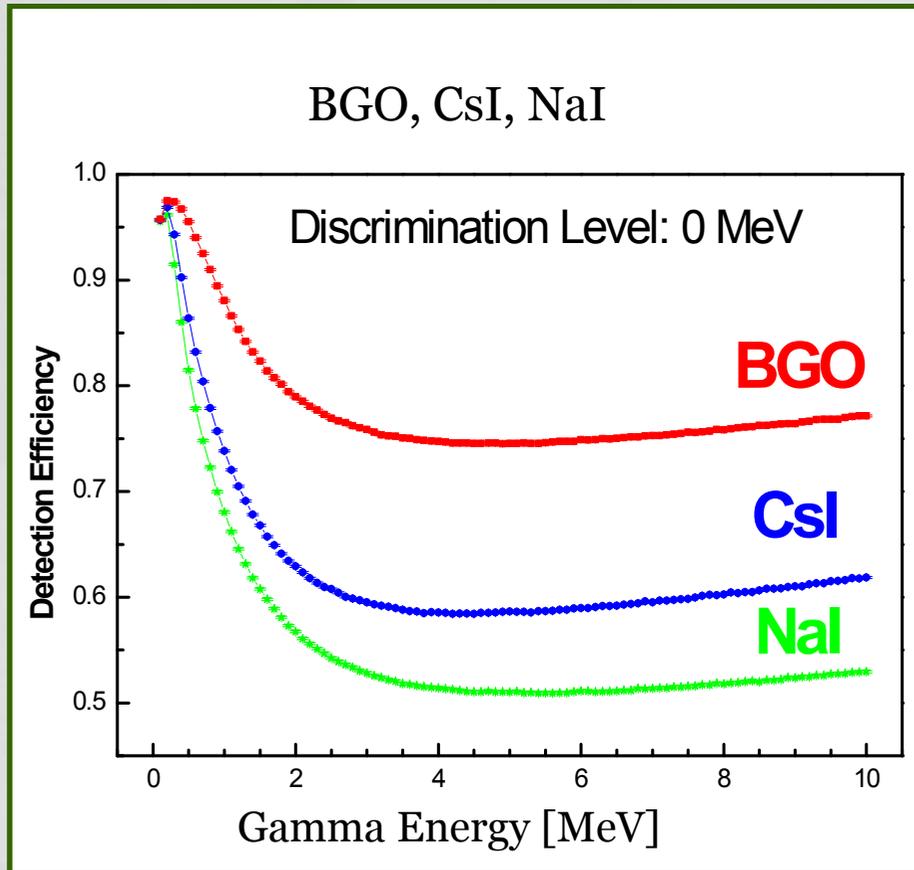
* The scintillator efficiency of NaI (Tl) is normalized to 100%

w4

in the simulation code, the physical processes include general physics process:electron magnetic process and optical process.em include photonelectron effect,compton effect and pair production,for electron ,there are ionization, multiscattering, anni.
wang, 2007-10-16

Detection Efficiency using Geant4

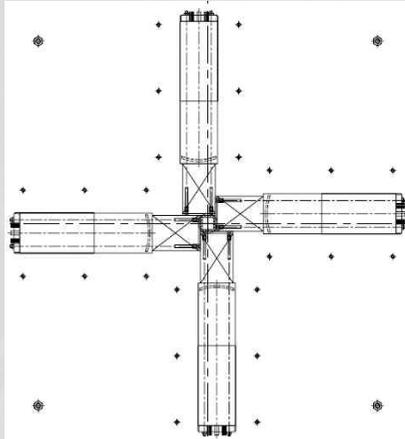
$$\text{Detection Efficiency} = \frac{N_{>\text{Threshold Energy}}^{\text{Accepted}}}{N_{\text{Total}}}$$



w5

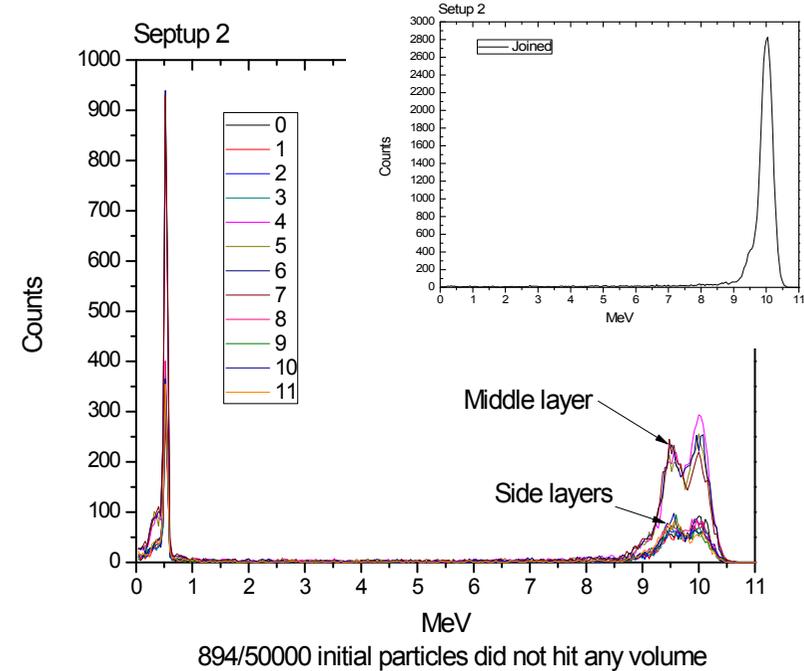
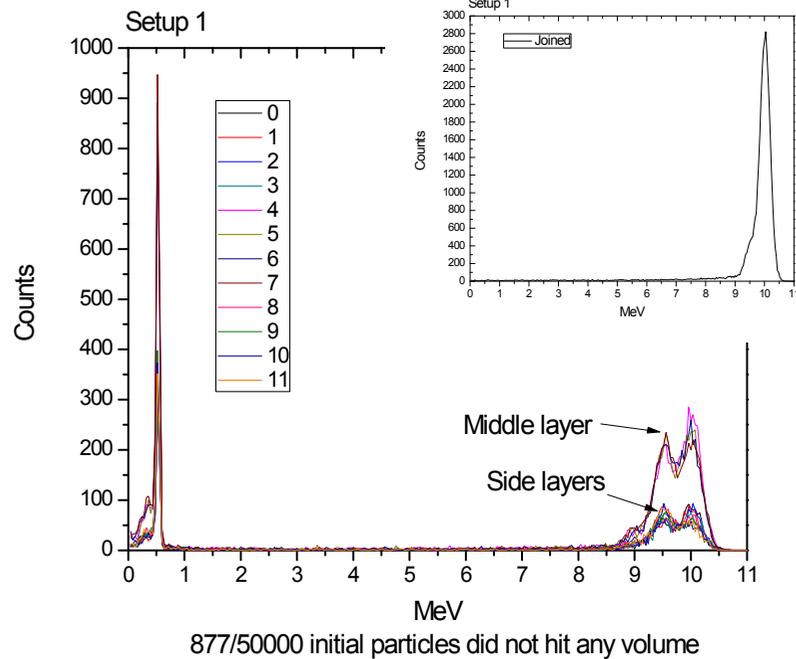
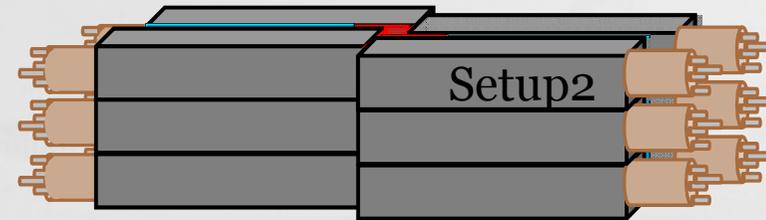
we simulated the total intrinsic efficiency of detector array for different crystals materials under 15MeV. from comparison the highest efficiency were given by BGO crystal case.
wang, 2007-10-17

4 π BGO Gamma Detector 1

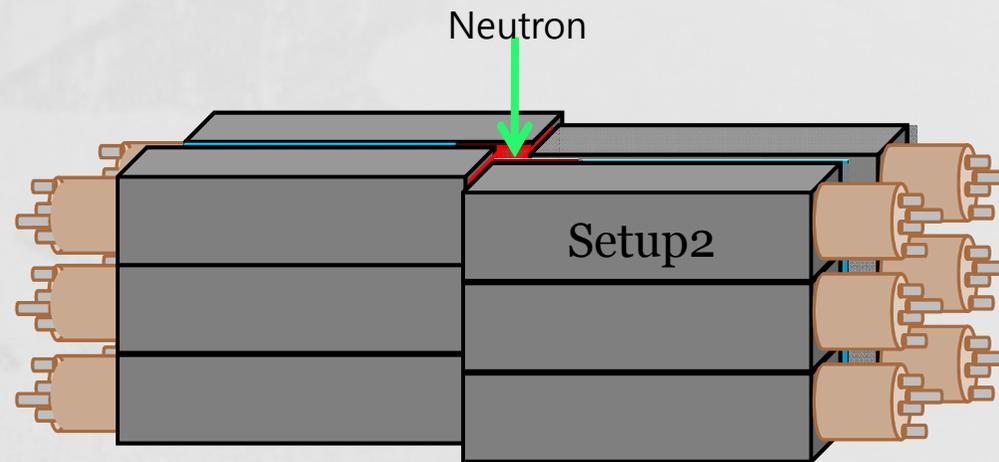
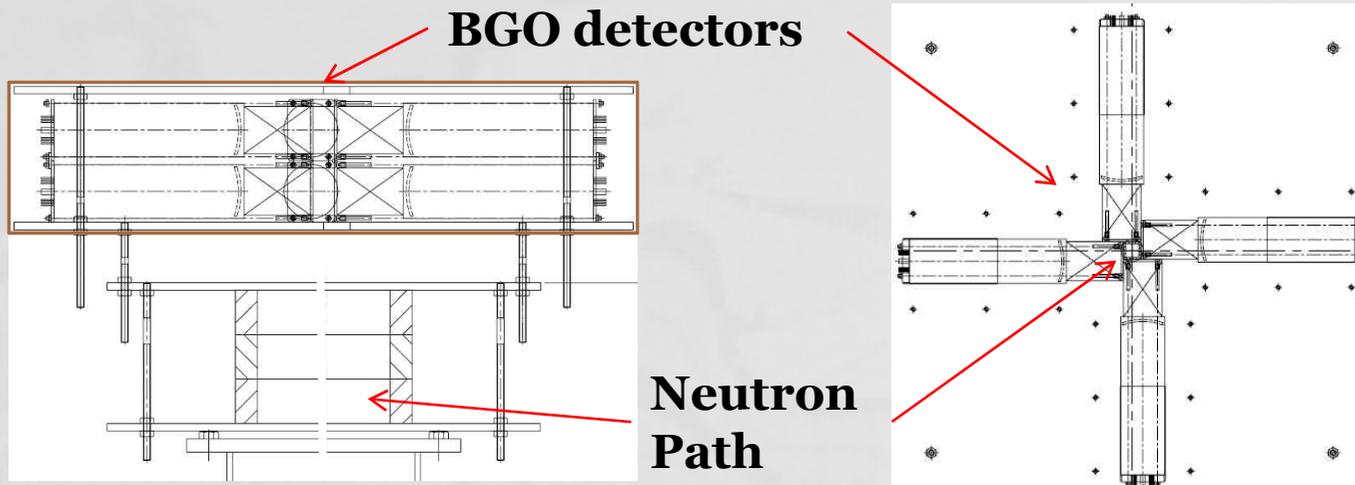


Setup1
98.25%
coverage

Setup2 : 98.21% coverage



4π BGO Gamma Detector 2

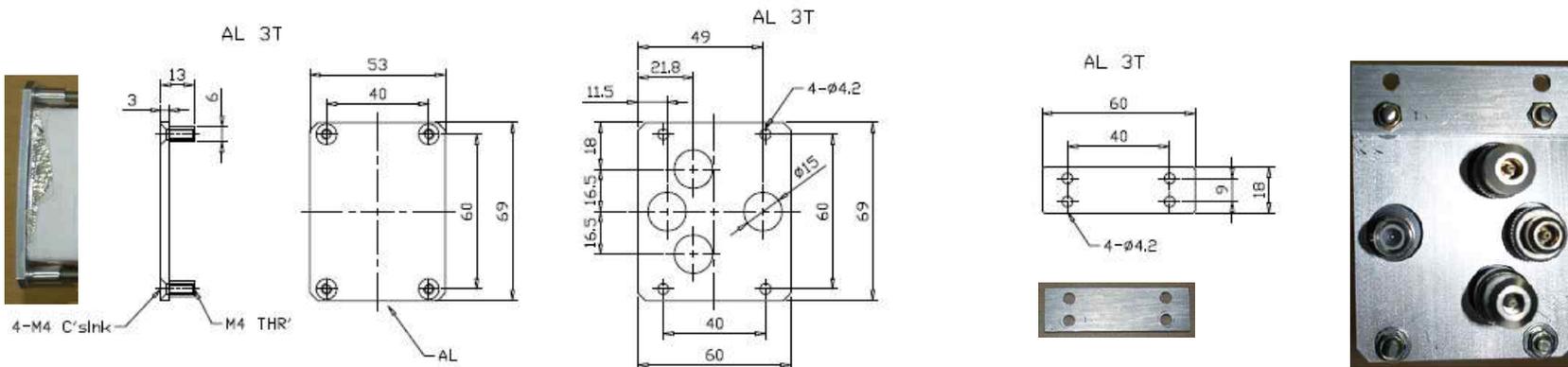
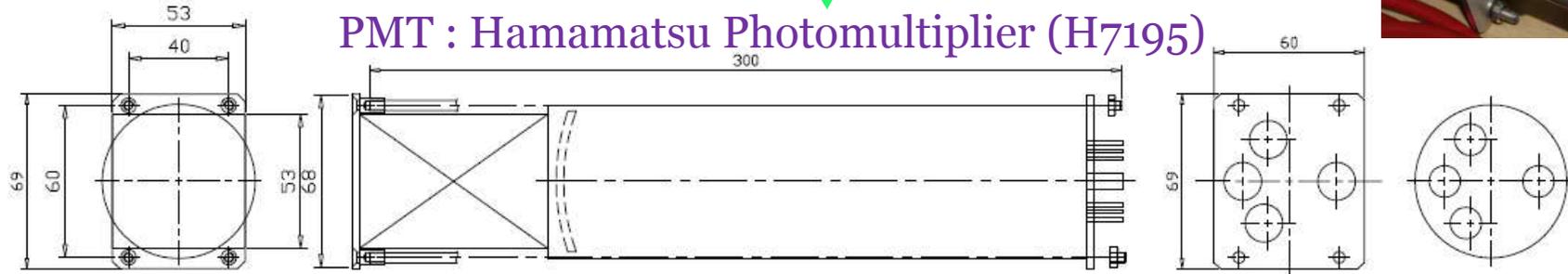


BGO detector modules 1

BGO made by SHANGHAI SICCAS HIGH TECHNOLOGY Corporation



PMT : Hamamatsu Photomultiplier (H7195)



BGO detector modules 2

Assembly Process of a module of BGO gamma detector

Preparation of
Module frame



Light Shield
For BGO



Cleaning BGO
And PMT



Optical
Connection

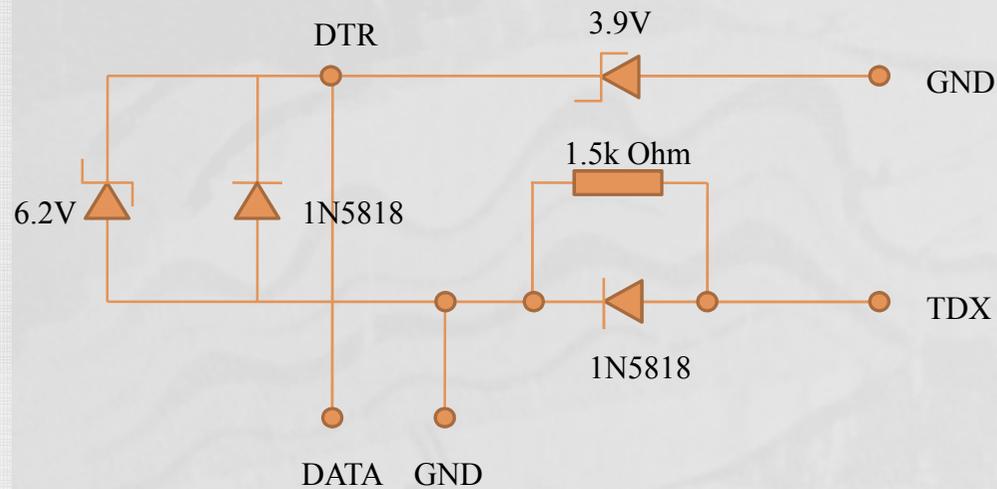


Fabrication
Of a Module



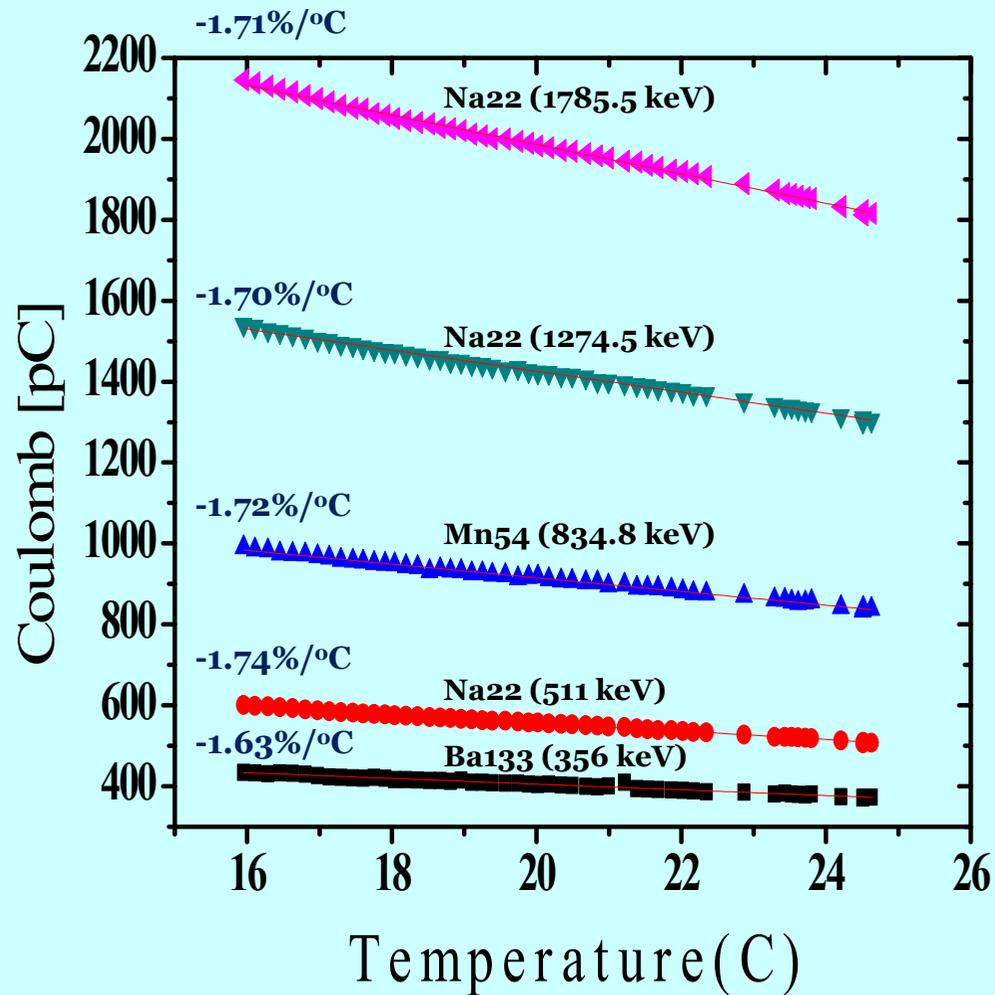
BGO Temperature & Energy 1

Temperature Sensor Using DS1820



Reference : Digital Temperature Sensor Module for Linux
<http://www.digitemp.com>

BGO Temperature & Energy 2



Our BGO Light Yield of Temperature Dependence From 16 °C to 26 °C

-1.7%/°C @ 16 °C
[-1.38%/°C @ 5 °C]

[REFERENCE]

Saint Gobain BGO :
[SGC_BGO_Data_Sheet.pdf]

-1.2%/°C

Naisen Zhang et al. :
[IEEE Trans. On Nuclear science
Vol 37, No 2, April 1990]
From 5 °C to 40 °C

-0.9%/°C @ 5 °C

**Temperature Calibration
Using Na22, Mn54, and Ba133**

BGO Temperature & Energy 3

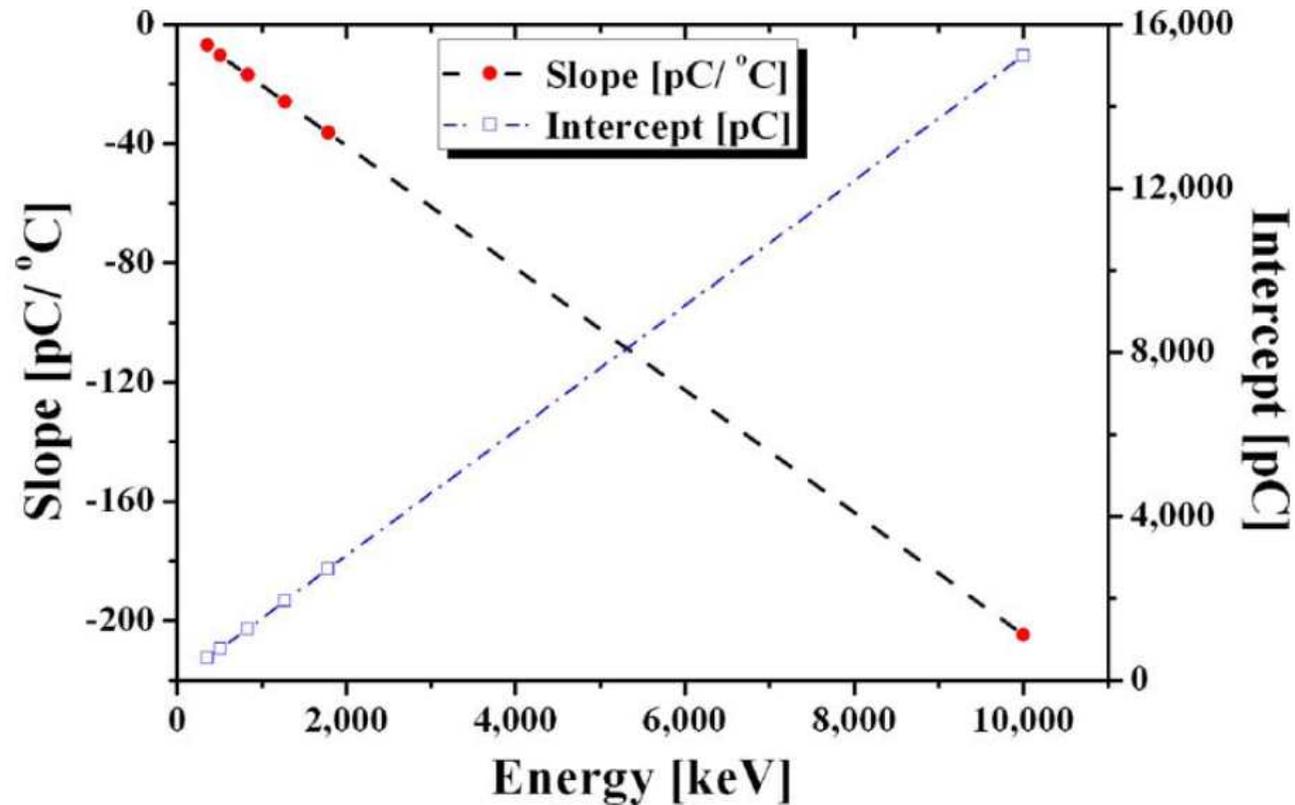
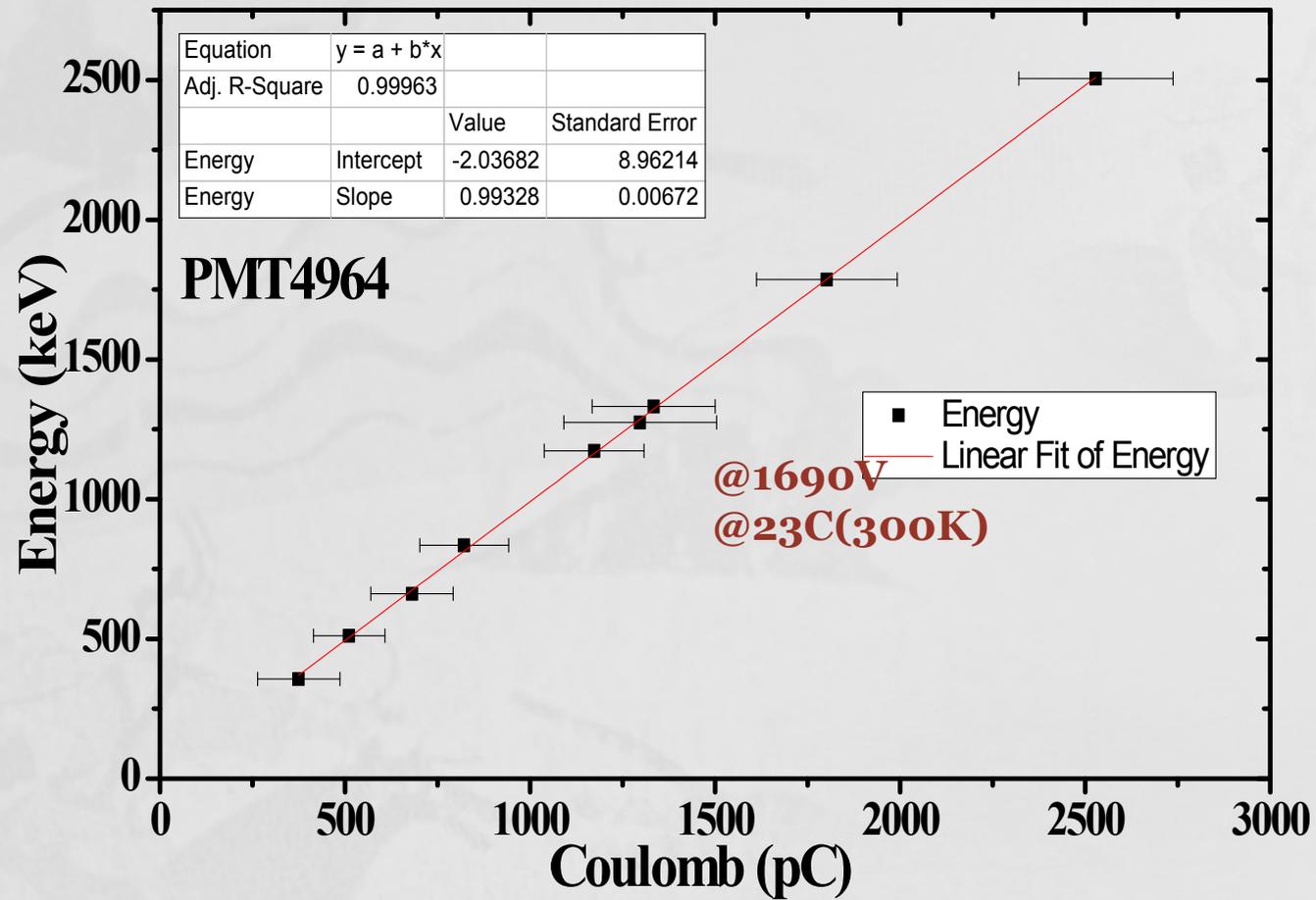


Fig. 7. Slope and the interception as a function of γ -ray energy.

Energy Linearity

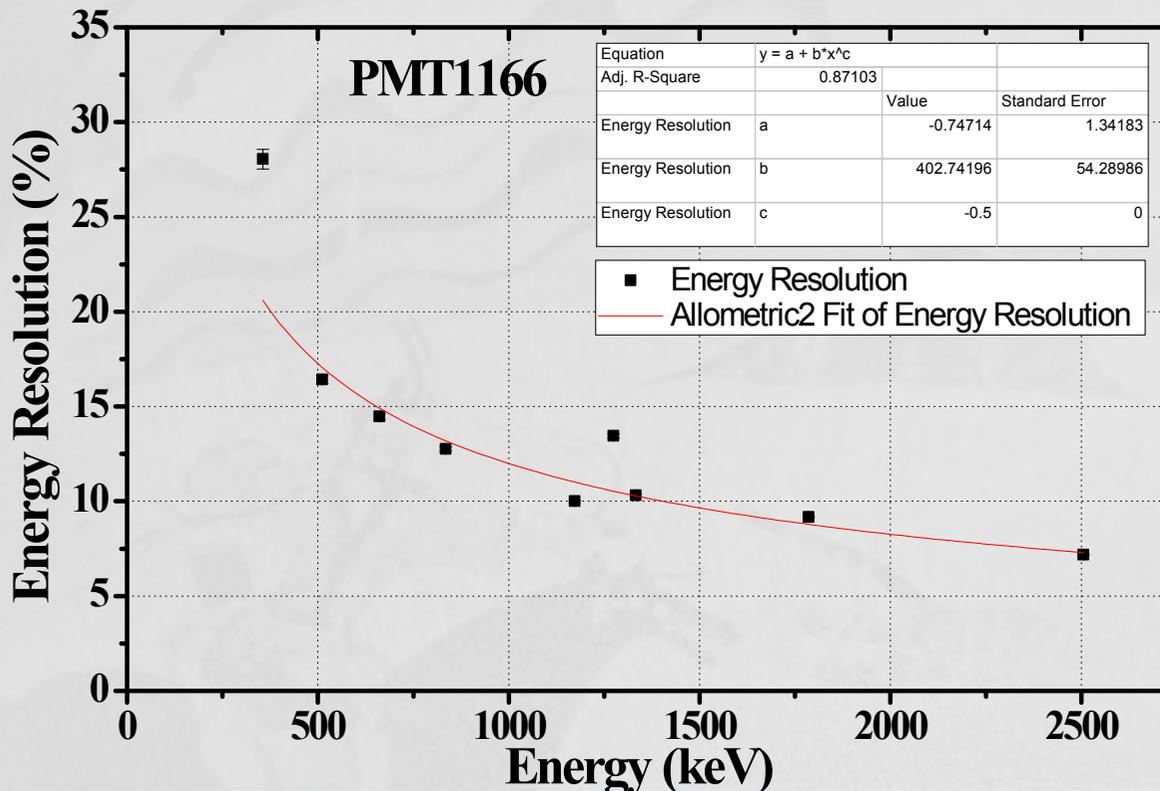
Energy Linearity compensated at 23°C



Energy Resolution

$$\frac{\Delta E}{E} [\%] = \frac{400}{\sqrt{E[\text{keV}]} - 0.7} = \frac{12.65}{\sqrt{E[\text{MeV}]} - 0.7}$$

~20% @ 400 keV, ~12% @ 1 MeV, ~8% @ 2.5MeV

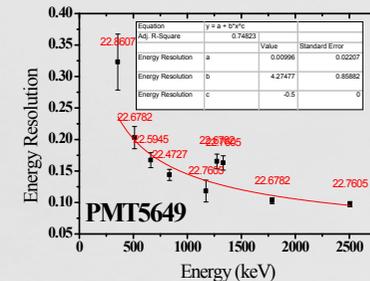
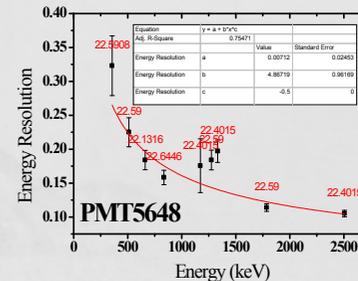
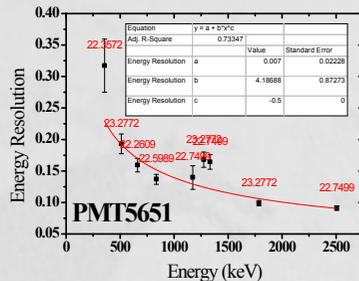
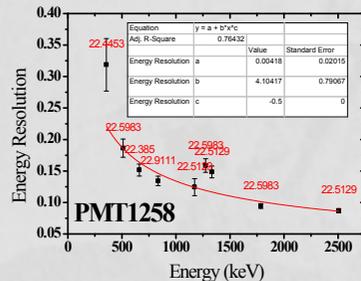
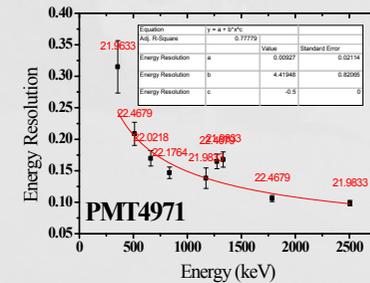
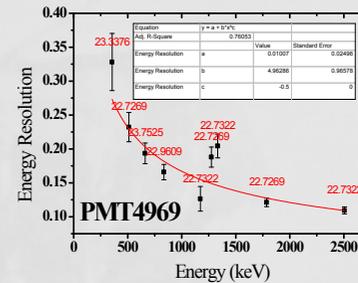
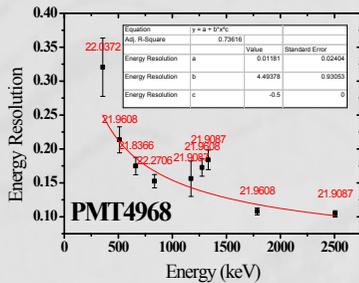
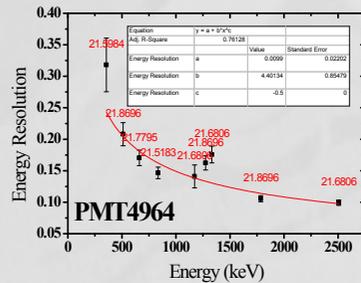
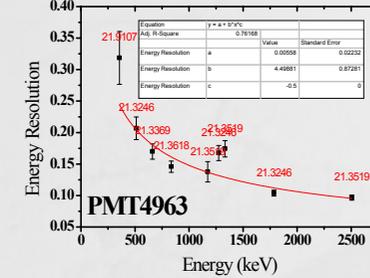
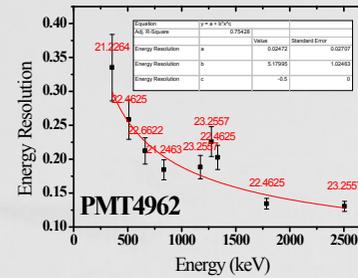
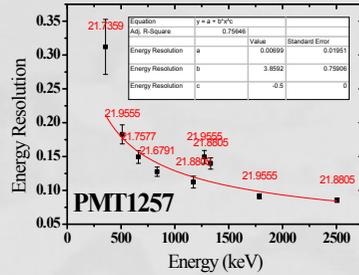
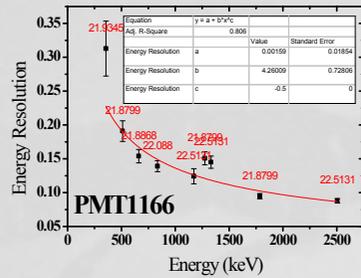


[REFERENCE]

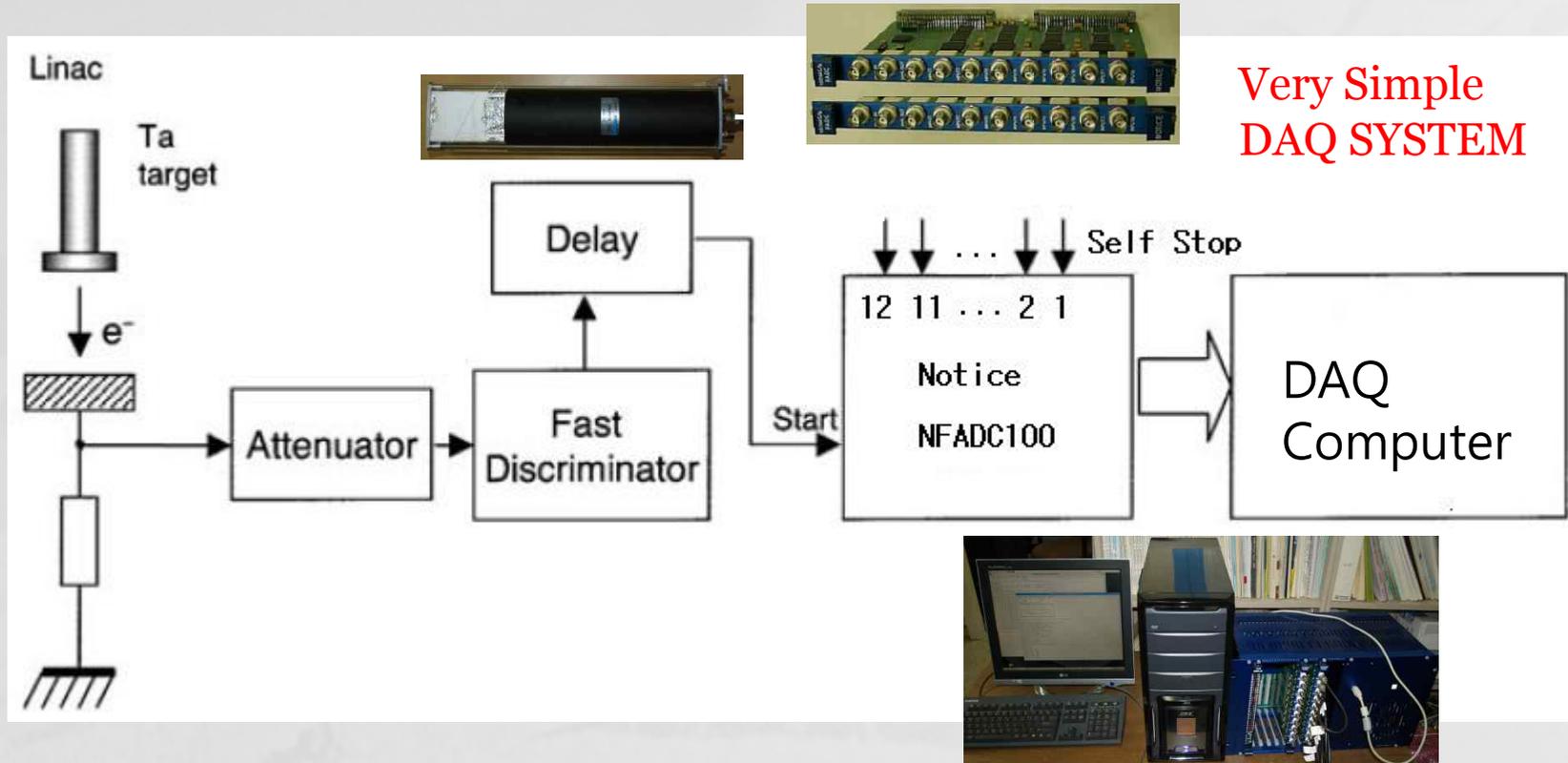
Naisen Zhang et al. :
 [IEEE Trans. On Nuclear science
 Vol 37, No 2, April 1990]

12% @ 1 MeV

Final BGO Module Test Result



4π BGO γ -detector FADC DAQ 1



- TOF spectra (TIME) + Pulse Height spectra (Signal Shape)
- Possible for Pulse shape analysis
- It needs fast computing system

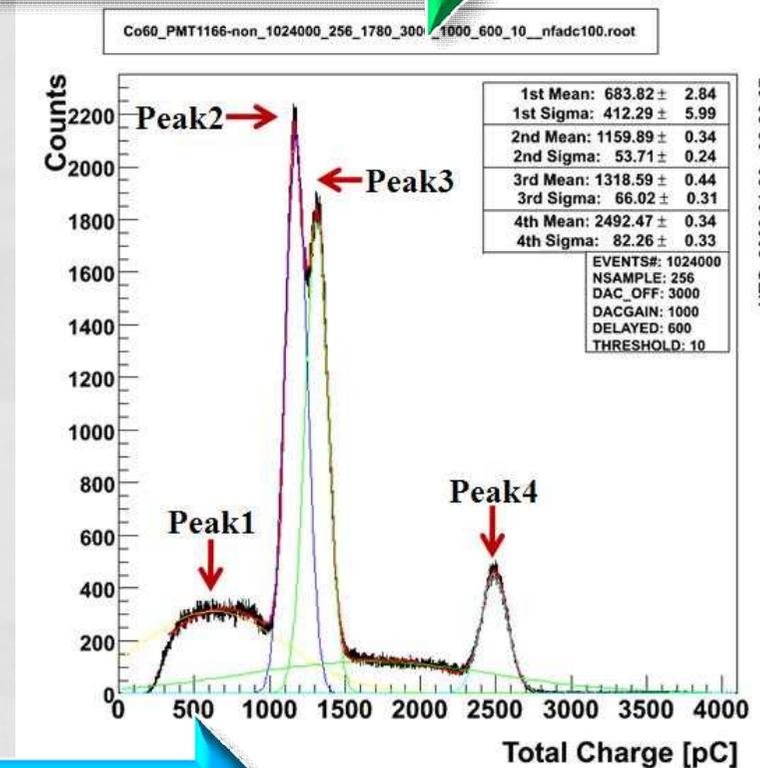
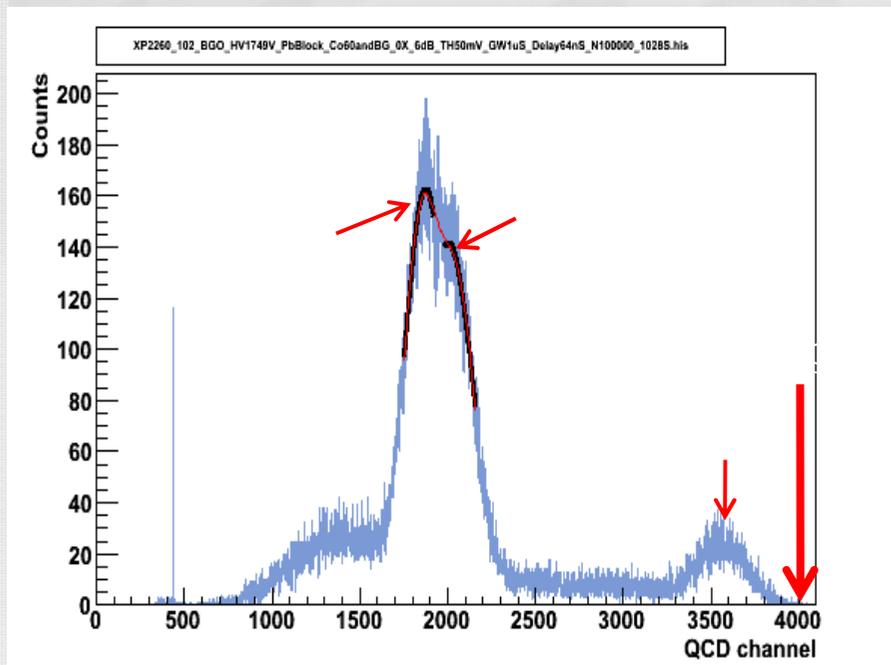
BGO γ -detector FADC DAQ 2

Different DAQ result using same BGO detector

VME ADC V792

Notice Korea 100 MHz FADC

More than 10 times better Charge Resolution



Better Peak Separation

Current Status

- 12 BGO detector modules are made and tested for Temperature, Linearity, and Resolution functions.
- Basic FADC DAQ system has been tested during Total Cross-section measurement and BGO module test.
- FADC DAQ system for Capture experiment will be arrived this year.
- The DAQ software to record 12 BGO signals will be tested next year.
- The on-line spectrum monitoring software will be developed next year.